Appendix A

INEEL Managers and Contractors

DOE-ID Managers

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Leonard E. "Bill" Johnston	4/49-4/54
Allan C. Johnson	4/54-12/61
Hugo N. Eskildson	1/62-11/63
William L. Ginkel (Acting)	11/63-3/64
William L. Ginkel	3/64-9/73
R. Glenn Bradley	9/73-3/76
Charles E. Williams	5/76-6/83
Troy E. Wade	7/83-6/87
Don Ofte	6/87-12/89
Phil Hamric (Acting)	1/90-4/90
Augustine Pitrolo	4/90-2/94
John Wilcynski (Acting)	2/94-10/94
John Wilcynski	10/94-2/99
Warren E. Bergholz, Jr. (Acting)	2/99-5/99
Beverly A. Cook	5/99-

Prime Operating Site Contractors

1950-1966	Phillips Petroleum Company
1966-1972	Idaho Nuclear Corporation (Allied Chemical Corporation, Aerojet General Corporation, and Phillips Petroleum Company)
1972-1976	Aerojet Nuclear Corporation
1976-1994	EG&G Idaho
1994-1999	Lockheed Martin Idaho Technologies Company
1999-	Bechtel BWXT Idaho, LLC
Argonne Natio	onal Laboratory-West
1949-	University of Chicago
Idaho Chemic	al Processing Plant (Idaho Nuclear Technology and Engineering Center)
1950-1953	American Cyanamid Company
1953-1966	Phillips Petroleum Company
1966-1971	Idaho Nuclear Corporation
19712-1979	Allied Chemical Corporation
1979-1984	Exxon Nuclear Idaho Company
1984-1994	Westinghouse Idaho Nuclear Company
1994-1999	Lockheed Martin Idaho Technologies Company
1999-	Bechtel BWXT Idaho, LLC
Specific Manu	facturing Capability
1983-1986	Exxon Nuclear Idaho Company
1986-1991	Rockwell INEL
1991-1994	Babcock & Wilcox Idaho Inc.
1994-1999	Lockheed Martin Idaho Technologies Company
1999-	Bechtel BWXT Idaho, LLC
Naval Reactor	s Facility
1950-1999	Westinghouse Electric Corporation
1999-	Bechtel Bettis, Inc.

APPENDIX B

Fifty Years of Reactors at the INEEL

fter the first reactor at the National Reactor Testing Station (Experimental Breeder Reactor-I) went critical in 1951, scientists built and operated dozens more reactors in the next five decades. Since the 1970s, it has become accepted that 52 reactors operated at the Site.

But counting reactors at a reactor research facility is not as straightforward as it might seem, nor is accumulating vital statistics about each reactor. While considering the reactors that operated on the Idaho desert, the following thoughts might be kept in mind.

First, scientists in different programs did not seem to follow the same rules when it came to naming reactors. For example, the Aircraft Nuclear Propulsion (ANP) program modified the core of the reactor it called HTRE-1 and named it HTRE-2. These were subsequently known as two reactors. In another program, experimenters changed the core of the Organic Moderated Reactor more than once, but the reactor retained the same name and was counted as one reactor. When the Experimental Breeder Reactor-II operated as a prototype of the Integral Fast Reactor, the name did not change. Thus, any list of reactors very definitely understates and under-represents the actual complexity of reactor development at the Site.

Second, the list-maker must decide what to commemorate in a list of reactors. Should reactors that never went critical be given a place? If so, the list will include the Experimental Organic Cooled Reactor. No uranium fuel ever was loaded into the reactor and it never operated or went critical before the program was canceled. It was "a reactor," but never "an operating reactor." This was true as well for the Experimental Beryllium Oxide Reactor.

This list does not include simulated reactors such as Semiscale, which was part of the reactor safety testing program. The omission of facilities like this is another way in which a list can understate the variety and complexity of the INEEL's nuclear reactor history.

Finally, not all the information one might desire about the history of a reactor is easily found. For example, one goal for this list was to identify the day, month, and year of initial criticality for each reactor—and the date of its final shutdown. But the INEEL Technical Library's vast collection of archived reports did not yield this information for each reactor. Some report writers were content to report that a reactor went critical "in the summer of" a certain year and leave it at that. The same writers may have considered other milestones, such as its first operation at "full power," to be more meaningful in the progress of their particular reactor.

This alphabetical list of reactors contains the names of 52 reactors (the fifty that operated and the two that did not) as they have been known traditionally, their acronyms, selected milestone dates, and descriptive information about each reactor. All references to megawatts are "thermal" megawatts. Readers who examine this list are invited to contribute additional milestone dates and other vital statistics about the reactors so that future lists might be made more complete and more accurate.

Na	ıme	Acronym	Reactor Startup	Last Day of Operation
1.	Advanced Reactivity Measurement Facility No. 1 The ARMF-I, a reactor located in a small pool in a building east of the MTR in the Test Reactor Area, was used to determine the nuclear characteristics of reactor fuels and other materials subject to testing in the MTR. Together with the MTR, the reactor helped improve the performance, reliability, and quality of reactor core components. Until the next generation reactor, the ARMF-II, this was considered the most sensitive device for reactivity determinations then in existence.	ARMF-I	10-10-60	1974
2.	Advanced Reactivity Measurement Facility No. 2 The ARMF-II was built in the opposite end of the tank occupied by ARMF-I. It had a "readout" system which automatically recorded measurements on IBM data cards. This refinement over the ARMF-I meant that operators could process data quickly in electronic computers. Designers of the ARMF-II benefitted from previous experience with the ARMF-I and the Reactivity Measurement Facility (described below).	ARMF-II	12-14-62	
3.	Advanced Test Reactor Located at the Test Reactor Area, the ATR, which continued to operate in 2000, is a materials testing reactor. It simulates the environment within a power reactor for the purpose of studying the effect of radiation on steel, zirconium, and other materials.	ATR	7-2-67	Continuing
	The ATR produces an extremely high neutron flux up to 1 x 10 ²⁵ neutrons per square centimeters per second. Target materials are exposed to the neutron flux for selected periods of time to test their durability within an environment of high temperature, high pressure, and high gamma radiation fields. Data that normally would require years to gather from ordinary reactors can be obtained in weeks or months in the ATR.			
	The ATR can operate at a power level of 250 megawatts. Its unique four-lobed design can deliver a wide range of power levels to nine main test spaces, or loops. Each loop has its own distinct environment apart from that of the main reactor core. Therefore, nine major experiments can take place simultaneously. Additional smaller test spaces surrounding the loops allow for additional tests.			
	In addition to materials testing, the ATR has made radioisotopes used in medicine, industry, and research.			
4.	Advanced Test Reactor Critical Facility The ATRC performs functions for the ATR similar to those of the ARMF reactors in relation to the MTR. It was a valuable auxiliary tool in operation long before the ATR startup. It verified for reactor designers the effectiveness of control mechanisms and physicists predictions of power distribution in the large core of the ATR.	ATRC	5-19-64	Continuing
	Low-power testing in the ATRC conserved valuable time so that the large ATR could irradiate experiments at high power levels. The ATRC is also used to verify the safety of a proposed experiment before it is placed in the ATR.			
5.	Argonne Fast Source Reactor The Argonne Fast Source Reactor was a tool used to calibrate instruments and to study fast reactor physics, augmenting the Zero Power Plutonium Reactor research program. Located at Argonne-West, this low-power reactor—designed to operate at a power of only one kilowatt—contributed to an improvement in the techniques and instruments used to measure experimental data.	AFSR	10-29-59	Late 70s
6.	Boiling Water Reactor Experiment No. 1 BORAX-I was a pioneer reactor that tested the safety and operating parameters of reactors which used boiling water as a moderator and coolant. In this reactor type, water is allowed to boil in the core. Saturated steam drives the turbines and generates power.	BORAX-I	1953	July 1954
	BORAX-I, like the later BORAX experiments, was located just north of EBR-I. It demonstrated that the boiling water moderated reactor concept was feasible for power reactors. Its design capacity was 1.4 megawatts. Operators destroyed it in July 1954 in a deliberately planned "destructive test," the purpose of which was to subject it to extreme operating conditions and learn more about the limits of its safe operation.			

Appendix B

Na	me	Acronym	Reactor Startup	Last Day of Operation
7.	Boiling Water Reactor Experiment No. 2 BORAX-II continued the testing program for boiling water reactors, this time at a power level capacity of 6 megawatts. Tests used fuels with varying enrichments of uranium-235.	BORAX-II	10-19-54	March 1955
8.	Boiling Water Reactor Experiment No. 3 The operating capacity of BORAX-III was 15 megawatts. The reactor was connected to a 2000-kilowatt turbine/generator set so that engineers could generate electricity, the ultimate objective of the reactor test program. On the night of July 17, 1955, the reactor produced sufficient power to light the city of Arco (500 kilowatts), the BORAX test facility (500 kilowatts), and part of the Central Facilities Area at the NRTS (1000 kilowatts).	BORAX-III	6-9-55	1956
9.	Boiling Water Reactor Experiment No. 4 BORAX-IV, with a power level of 20 megawatts, tested fuel elements made from mixed oxides (ceramics) of uranium and thorium. These materials had a high capacity to operate in the extreme heat of a reactor before they failed.	BORAX-IV	12-3-56	June 1958
	The ceramic core demonstrated that a reactor loaded with this fuel could operate safely and feasibly. The fuel could operate in higher temperatures, was less reactive with the water coolant in case the cladding ruptured, was cheaper to manufacture, and burned a larger percentage of the fuel before loosing its reactivity. The reactor produced measurable quantities of the artificial thorium-derived fuel, uranium-233. One series of BORAX-IV tests involved operating the reactor with experimentally defective fuel elements in the core.	or		
10.	Boiling Water Reactor Experiment No. 5 BORAX-V could operate at a power level of 40 megawatts. This flexible reactor advanced the boiling water reactor concept by testing the safety and economic feasibility of an integral, nuclear superheat system. On October 10, 1963, it produced superheated (dry) steam entirely by nuclear means for the first time. The reactor demonstrated that improved efficiency from manufactured steam is obtainable by incorporating as a design feature a number of superheated fuel assemblies in the reactor core lattice.	BORAX-V	2-9-62	September 1964
11.	Cavity Reactor Critical Experiment Located at TAN, CRCE was an outgrowth of a program begun by NASAin the 1960s to investigate the propulsion of space rockets by nuclear power, offering the possibility of much greater thrust per pound of propellant than chemical rockets. The concept for the cavity reactor core was that the uranium would be in a vapor, or gaseous, state. Hydrogen propellant flowing around it would theoretically attain much higher temperatures—up to 10,000° F—than in conventional solid core rockets. The experiments at TAN used simulated hydrogen propellant and produced data on the reactor physics feasibility of a gaseous core being able to go critical.	CRCE	5-17-67	Early 1970s
	The core was uranium hexafluoride (UF $_6$); the experiments were all done at the relatively low temperature of about 200° F. In the proposed ultimate application, the ball of uranium gas would be held in place by the hydrogen flowing around it, something like a ping-pong ball suspended in a stream of air. Uranium core temperatures as high as 100,000° F were considered possible.			
12.	Coupled Fast Reactivity Measurement Facility When the ARMF-II reactor was modified in 1968, it was given a new name, the CFRMF. A section of the core was modified to produce a region of high-energy neutron flux useful in comparing calculated and observed results. This tool provided physics information about the behavior of fast (ie, unmoderated) neutrons Physicists studied differential cross sections and tested calculational methods. The CFRMF contributed to the development of fast neutron reactors.	CFRMF	1968	1991
13.	Critical Experiment Tank The CET reactor produced a source of neutrons used to calibrate various types of neutron sensors and chambers. Part of the Aircraft Nuclear Propulsion program and located at Test Area North (TAN), the CETwas a low-power reactor (one of three in the ANP program) originally designed to mock-up the HTRE-I and HTRE-II reactors. Later fuel test bundles intended for testing in HTRE-II were first evaluated for reactivity characteristics in the CET.	CET	1958	1961

Name	Acronym	Reactor Startup	Last Day of Operation
14. Engineering Test Reactor When the Engineering Test Reactor started up at the TRAin 1957, it was the largest and most advanced materials test reactor in the world. The 175-megawatt reactor provided larger test spaces than the older MTR and provided a more intense neutron flux. The ETR evaluated fuel, coolant, and moderator material under environments similar to those of power reactors.	ETR s	9-19-57	December 1981
In 1972 the ETR was modified by the addition of a Sodium Loop Safety Facility into the reactor core. With this, the reactor played a new role supporting DOE's breeder reactor safety program. ETR test programs related to the core design and operation of breeder reactors. As testing progressed, the reactor was again modified with a new top closure accommodating the irradiation loop. Other additions included a helium coolant system and sodium-handling system. The ETR was the first complete reactor facility to be deactivated and documented immediately after shutdown.			
15. Engineering Test Reactor Critical Facility ETRC was a full-scale, low-power nuclear facsimile of the ETR, similar in function to the ARMF and ATRC. It was used to determine in advance the nuclear characteristics of experiments planned for irradiation in ETR and the power distribution effects for a given ETR fuel and experiment loading. Since no two ETR loadings were identical, the ETRC allowed operators to predict the ETR's nuclear environme when completed experiments were removed or new ones added.	ETRC	5-20-57	1982
This information was necessary to calculate the experiment irradiation and determine core life, control roc withdrawal sequences, reactivity worths, and core safety requirements.	i		
Proposed fuel and experiment loadings were first mocked up in ETRC and manipulated until a desired power distribution throughout the core was attained, satisfying pertinent safety requirements. The ETRC's low-power tests allowed the ETR to operate without interruption, saving time and money.	3		
16. Experimental Beryllium Oxide Reactor Modifications of a former ANPbuilding at TAN, the Shield Test Pool Facility, began in May 1963 to hous the Experimental Beryllium Oxide Reactor (EBOR). The objective of the reactor was to develop beryllium oxide as a neutron moderator in high-temperature, gas-cooled reactors. The project was canceled in 1966 before construction was complete.		Never ope	rated
Among the reasons for the cancellation was the encouraging progress achieved, concurrent with EBOR construction, in developing graphite as a moderator. This reduced the importance of developing beryllium oxide as an alternate.			
17. Experimental Breeder Reactor No. I EBR-I, the first reactor built at INEEL, began operation in 1951. The reactor produced the first usable electricity from nuclear heat on December 20, 1951. It achieved full-power operation the next day. In 195 the reactor confirmed that a nuclear reactor designed to operate in the high-energy neutron range is capable of creating more fuel than its operation consumes ("breeding").		8-24-51	12-30-63
The reactor, which used enriched uranium as fuel, was unmoderated. It used sodium-potassium alloy (NaK) coolant. Ablanket of uranium-238 around the core provided the "fertile" material in which breeding took pla The liquid-metal coolant permitted the neutron energies to be kept high, thus promoting fissionable-mater breeding. The coolant also enabled high-temperature and low-pressure operation, both conducive to efficie power production.	ace. ial		
President Lyndon B. Johnson dedicated EBR-I as a National Historic Landmark on August 26, 1966. It was subsequently opened to the public for visits and tours.	as		
18. Experimental Breeder Reactor No. II Part of the continuing investigation of fast neutron breeding reactors, the EBR-II, located at Argonne-Wes inside a containment shell, was built to demonstrate the feasibility of on-site fuel reprocessing as an adjunction of the containment shell.		9-30-61	9-30-94

Appendix B

Name	Acronym	Reactor Startup	Last Day of Operation
Experimental Breeder Reactor No. II (continued) to a liquid metal-cooled fast-breeder-reactor power plant. These objectives were met within the first few years of its operation.			
By September 1969, EBR-II operated at a capacity of 62.5 megawatts and supplied electricity to Argonne-We and the power grid at the Site until its shutdown in 1994. The reactor operated submerged in a tank of liquis sodium coolant. During recycle experiments between 1964-1969, spent fuel was sent by automated handlin methods to the Fuel Cycle Facility adjacent to the reactor building, treated by pyrometallurgical techniques and the useful fissile metal refabricated into new fuel pins.	d g		
EBR-II also was used to irradiate reactor fuel and structural material samples, testing their durability in breeder-reactor environments. This information helped improve fuel and material performance for future breeder reactors.			
19. Experimental Organic Cooled Reactor Because the OMRE (see below) was built as a minimum-cost facility (\$1,800,000) to test the feasibility of the organic-cooled reactor concept, it lacked test loops needed to investigate various organic coolants and experimental fuel elements. The EOCR was intended to extend and advance the OMRE studies.	EOCR	Never ope	rated
During the final stages of its construction, EOCR was placed in standby status in December 1962 when the AEC decided that the organic-cooled concept would not significantly improve nuclear power plant performance over what other reactor concepts already had achieved. The building, located east of the Central Facilities Area, was recycled for other (non-nuclear) uses.			
20. Fast Spectrum Refractory Metals Reactor This low-power critical facility operated at TAN to collect data for a proposed fast-spectrum refractory-met reactor concept called the 710 Reactor. The concept involved using metals such as tungsten and tantalum in a compact, very high-temperature reactor for generating power in space.		March 196	52 1968
21. Gas Cooled Reactor Experiment Built at the Army Reactor Experimental Area, later called the Army Reactor Area (ARA), the GCRE was a water-moderated, nitrogen (gas)-cooled, direct- and closed-cycle reactor. It generated 2,200 kilowatts of heat, but no electricity. The U.S. Army wanted to develop a mobile nuclear power plant, and the GCRE was the first phase of the program, proving the principle of this reactor concept. The reactor provided engineering and nuclear data for improved components. The GCRE was also used to train military and civilian personnel in operation and maintenance of gas-cooled reactor systems.		2-23-60	4-6-61
22. Heat Transfer Experiment No. 1 Test Area North was opened in 1952 for the Aircraft Nuclear Propulsion program, which operated during the 1950s to develop for the U.S. Air Force a nuclear-powered jet airplane using direct-cycle heat transfer engineering. The program involved ground tests only, but proved the principle of nuclear-powered turbojet engine operation with a full-power test in January 1956, with Heat Transfer Reactor Experiment No. 1 (HTRE which produced 20 megawatts of heat energy on a test stand at TAN's Initial Engine Test Facility. The water-moderated reactor used enriched uranium fuel clad in nickel-chromium.		11-4-55	1956
23. Heat Transfer Experiment No. 2 In order to irradiate fuel elements that were too large to fit in the MTR for materials tests, the ANP program drilled a hexagonal hole in the center of HTRE-1 and renamed it HTRE-2, converting it to a materials test reactor and subjecting test fuels to environments reaching 2,800° F. The ANP materials test program advanced the technology of high-heat ceramic reactor fuels.	HTRE-2	July 1957	3-28-61 (End of ANP program)

Name	Acronym	Reactor Startup	Last Day of Operation
24. Heat Transfer Experiment No. 3 After substantial testing and experimentation, this new experiment arranged the reactor, engine, shielding, and heat transfer systems in a horizontal configuration anticipating final design in an airframe.	HTRE-3	1958	December 1960
President John F. Kennedy canceled the ANPprogram on March 28, 1961. Work on the project came to an abrupt and permanent end on that date. The two HTRE experiments were moved to the site of EBR-I, where they are on display at the visitors center.(See Heat Transfer Experiment No. 1)			
25. High Temperature Marine Propulsion Reactor The 630-Areactor, a low-power critical experiment, was operated at TAN to explore the feasibility of an air-cooled, water-moderated system for nuclear-powered merchant ships. Further development was discontinued in December 1964 when decisions were made to lower the priority of the entire nuclear power merchant ship program.	630-A	1962	1964
26. Hot Critical Experiment HOTCE was an elevated-temperature critical experiment designed to obtain information on temperature coefficients of solid-moderated reactors, to develop a theory consistent with this information, and to develop measurement techniques for high-temperature reactors. A part of the ANPprogram, it operated in the Critical Experiment cell of the Low Power Test Facility at TAN. HOTCE was one of three low-power reactors supporting the ANP program, along with the Shield Test Pool Facility Reactor (see below) and the Critical Experiment Tank (CET). The ANP program ended in 1961.	НОТСЕ	1958	3-28-61 (End of ANP program)
27. Large Ship Reactor A 28. Large Ship Reactor B The A1W (aircraft carrier, first prototype, Westinghouse) plant consisted of a pair of prototype reactors for USS Enterprise, a U.S. Navy nuclear-powered aircraft carrier. Located at the Naval Reactors Facility, the two pressurized-water reactors (designated A and B) were built within a portion of a steel hull. The plant simulated Enterprise's engine room. All components could withstand seagoing use.	A1W-A A1W-B	10-21-58 July 1959	1-26-94 1987
The A1W plant was the first in which two reactors powered one ship propeller shaft through a single-geared turbine propulsion unit. As the Navy program evolved, new reactor cores and equipment replaced many of the original components. The Navy trained naval personnel at the A1W plant and continued a test program to improve and further develop operating flexibility.	2		
29. Lost of Fluid Test Facility The LOFTreactor, located at TAN within a containment building, was a centerpiece in the safety testing program for commercial power reactors. The reactor was a scale-model version of a commercial pressurized-water power plant built chiefly to explore the effects of loss-of-coolant accidents (LOCAs).	LOFT	1973	7-9-85
Thirty-eight nuclear power tests were conducted on various accident scenarios, including the real accident at Three Mile Island, between 1978 and 1985. Among other goals, the program investigated the capability of emergency core cooling systems to prevent core damage during a LOCA. Experiments at LOFT simulated small medium-, and large-break LOCAs, sometimes complicated with other events such as "loss of offsite power."	-,		
LOFT was inactivated in 1986, following completion of the LP-FP-2 experiment, the most significant severe- fuel-damage test ever conducted in a nuclear reactor. This test, which involved the heating and melting of a 100-rod experimental fuel bundle, provided information on the release and transport of fission products that could occur during an actual commercial reactor accident where core damage occurs.			
30. Materials Test Reactor The MTR was the original reactor at the Test Reactor Area and the second reactor operated at the NRTS. Fueled with enriched uranium fuel, water-cooled and -moderated, the reactor was a key part of the Atomic Energy Commission's post-war reactor development program. It supplied a high neutron flux in support of a	MTR	3-31-52	4-23-70

reactor development program subjecting potential reactor fuels and structural materials to irradiation. In

Appendix B

Name	Acronym	Reactor Startup	Last Day of Operation
Materials Test Reactor (continued) addition, its "beam holes" made it possible to perform cross-section and other physics research.			
The high-flux radiation fields available in this reactor made it possible to accelerate the screening of potent reactor materials. In its early years, the MTR contributed to the design of pressurized water, organic-model liquid-metal-cooled, and other reactors. Successful operation of the MTR itself was a great experiment rest in a family of plate-type reactors. The reactor operated at a power level of 30 megawatts until September 1 when thermal output was increased to 40 megawatts.	ated, ılting		
The MTR logged more than 125,000 operating hours and more than 19,000 neutron irradiations. During At 1958, the MTR became the first reactor to operate using plutonium-239 as fuel at power levels up to 30 megawatts. The demonstration showed that a plutonium-fueled reactor could be controlled satisfactorily.	ıgust		
The materials testing workload of the MTR was taken over by the new and larger Advanced Test Reactor.			
31. Mobile Low-Power Reactor No. 1 Following the operation of the GCRE, the ML-1 was the next major step toward the development of a moblow-power power plant for the U.S. Army. The entire ML-1 plant was designed to be transported either by standard cargo transport planes or standard Army low-bed trailers in separate packages weighing less than 40 tons each.	ML-1 ile	3-30-61	5-29-64
The reactor was operated remotely at the ARA-IVarea from a control cab at a distance of approximately 500 feet. It could be moved after a 36-hour shutdown. The reactor was designed for ease of operation and maintenance by enlisted technicians at remote installations, for reliable and continuous operation under extreme climatic conditions, and for the rigors of shipment and handling under adverse conditions.			
The ML-1 shut down for the last time after operating for a total of 664 hours. Before the ML-1 had reacher all of its performance goals, the Army phased out its reactor development program around 1965.	d		
32. Natural Circulation Reactor The S5G (submarine reactor, 5th prototype, General Electric) was the prototype of a pressurized-water reactor for <i>USS Narwhal</i> . Located at the Naval Reactors Facility, it was capable of operating in either a forced or natural circulation flow mode. In the natural mode, cooling water flowed through the reactor by thermal circulation, not by pumps. Use of natural circulation reduced the noise level in the submarine.	S5G	9-12-65	5-1-95
To prove that the design concept would work in an operating ship at sea, the prototype was built in a submarine hull section capable of simulating the rolling motion of a ship at sea. The S5G continued to operat as part of the Navy's nuclear training program until that program was reduced after the end of the Cold Wa			
33. Neutron Radiography Facility The NRAD, located in the Hot Fuel Examination Facility at Argonne-West, is a nondestructive examinatio tool. Using two collimated neutron beams produced by a 250-kilowatt reactor, NRAD produces neutron radiographs showing the internal condition of highly irradiated test specimens without physically cutting into the specimen. The reactor also has been used as a neutron source for isotope production, activation analysis, and the evaluation of radiation effects on materials.	NRAD n		Continuing
34. Nuclear Effects Reactor The Nuclear Effects Reactor (FRAN) was a small-pulsed reactor, capable of supplying bursts of high-intensity fast neutrons and gamma radiation. FRAN was transferred to the NRTS in mid-1967 from the Nevada Test Site, where it had been operated by Lawrence Livermore National Laboratory.	FRAN	8-28-68	June 1970
Located in the ARAbuilding formerly occupied by the ML-1 reactor, FRAN was used for a short time to test the performance of new detection instruments then being developed for reactor control purposes. The reactor was moved back to DOE's Lawrence Livermore National Laboratory in June 1970.			

Name	Acronym	Reactor Startup	Last Day of Operation
35. Organic Moderated Reactor Experiment OMRE demonstrated the technical and economic feasibility of using a liquid hydrocarbon as both coolant and moderator, a reactor concept developed and partially financed by Atomics International. Located a few miles east of the Central Facilities Area, the reactor operated with a succession of cores. The waxy coolant was considered promising because it liquified at high temperatures but didn't corrode metal like water did. Also, it operated at low pressures, significantly reducing the risk of leaking. A scaled-up reactor, the Experimental Organic Cooled Reactor, was built next door in anticipation of further development of the concept	OMRE	9-17-57	April 1963
36. Power Burst Facility Located southeast of the Test Reactor Area, the PBF was part of the reactor safety testing program. It was designed to simulate various kinds of imagined accidents caused by sudden increases in the operating level of a reactor. The PBF was the only reactor in the world that could perform rapid power changes (bursts) within milliseconds. It performed severe-fuel-rod-burst tests and also simulated loss-of-coolant accidents within a special assembly that fit inside the main reactor core.	PBF	9-22-72	1985
The initial mission for PBF was to test light water reactor fuel rods under representative accident conditions. Data from these tests were used to develop and validate fuel behavior computer codes for the Nuclear Regulatory Commission.			
After its test program ended in 1985, the PBF reactor was considered for use in defense-related programs or for use in a brain cancer treatment program called Boron Neutron Capture Therapy (BNCT). The BNCT program would have treated patients with glioblastoma multiforme—a form of brain cancer. However, neither of these missions materialized.			
37. Reactivity Measurement Facility RMF, a detector reactor that measured reactivity changes in materials irradiated in the MTR or ETR, was operated for more than eight years. The RMF was used to assay new and spent fuel elements and to assist in experiment scheduling by evaluating reactivity losses and flux depression caused by in-pile apparatus.	RMF	2-11-54	4-10-62
38. Shield Test Pool Facility The SUSIE reactor was used for bulk shielding experiments that were performed in support of the ANP Shielding Experimentation Program. The reactor, situated in a water-filled pool at TAN, could be operated safely, was adaptable to many forms of nuclear research, and was easy to operate at minimum cost. After the ANPprogram was discontinued in 1961, SUSIE continued in use by other programs at the NRTS.	SUSIE	1961	
39. Special Power Excursion Reactor Test No. I SPERT-I was the first in a series of four safety-testing reactors designed to study the behavior of reactors when their power level changed rapidly. Power runaways were produced deliberately by moving the control rods. The variables in the thousands of SPERT studies included fuel plate design, core configuration, coolant flow, temperature, pressure, reflectors, moderators, and void and temperature coefficients.	SPERT-I	6-11-55	1964
All operations were conducted from a control building located a half mile from the reactors, situated a few miles east of the Central Facilities Area. SPERT-I was an open-tank, light-water-moderated and reflected reactor originally using 92 percent enriched uranium fuel. The reactor tank, about 4 feet in diameter and 14 feet high, was filled with water to a level about 2 feet above the core.			
In general, SPERT-I tests demonstrated the damage-resistant capabilities of low-enrichment (4 percent enriched uranium-235) uranium-oxide fuel pins similar to those used in water-cooled reactors powering large central statio	ns.		
40. Special Power Excursion Reactor Test No. II This facility consisted of a closed pressurized water reactor with coolant flow systems designed for operation with either light or heavy water. The pressure vessel was 24 1/2 feet high by 10 feet inside diameter. Tests with	SPERT-II	3-11-60	October 1964

heavy water (deuterium, an isotope of hydrogen) were desired because heavy water reactors were of growing

Appendix B

Name		Acronym	Reactor Startup	Last Day of Operation
Special Power Excursion Reactor Test No. II (continue importance in Canada, Europe, and the United States. various types of physics calculations on the effects of	Also, heavy water tests allowed for the verification of			
but was built second) provided the widest practical rang	et developed for studying the inherent safety was planned as the third in the series of SPERTreactors, ge of control over three variables: temperature, pressure, l similar to those used in commercial power production.	SPERT-III	12-19-58	June 1968
42. Special Power Excursion Reactor Test No. IV SPERT-IV was an open-tank, twin-pool facility that per by varying conditions including forced coolant flow, we head, and other hydrodynamic effects. The reactor, we aluminum alloyed, plate-type fuel elements.	ermitted detailed studies of reactor stability as affected ariable height of water above the core, hydrostatic	SPERT-IV	7-24-62	August 1970
The SPERT-IV facility was modified by the installation fuel samples to be inserted into a test hole in the center short-period excursions without damaging the "driver" destructive mechanisms continued until the Power But	r of the reactor core, where it could be subjected to 'fuel in the rest of the core. The CDC work on fuel-			
43. Spherical Cavity Reactor Critical Experiment SCRCE was the final experiment in reactor physics w the feasibility of a reactor going critical with a gaseou a cylindrical configuration because of its ease of const culmination of the project, allowing for a comparison shape was considered a more likely geometry for the	ork for the NASA-sponsored program to determine is core of uranium. Previous work had been done with ruction. The spherical configuration was the between theory and experimental results. The spherical	SCRCE	November 1972	1973
44. Stationary Low-Power Reactor (Earlier name - Ar The SL-1 reactor, originally named Argonne Low Pow as a prototype of a low-power, boiling-water reactor p The SL-1 was accidentally destroyed and three men k	ver Reactor (ALPR), was designed for the U.S. Army lant to be used in geographically remote locations.	SL-1, ALPR	8-11-58	1-3-61
45. Submarine Thermal Reactor With the S1W, also known as the Submarine Thermal R The purpose of a nuclear-powered submarine was to that could remain underwater powered by a fuel which	eactor (STR), the United States'nuclear navy was born. ransform submarines into "true submersibles," vessels	S1W, STR	3-30-53	10-17-89
Reactors Facility. Cooled and moderated by pressurized were installed inside two hull sections duplicating the s	nclear power plant was the first prototype built at the Naval water, the reactor and its associated propulsion equipmentize and specifications of <i>USS Nautilus</i> , under construction research, the hull sections were placed in a tank of water	nt 1		
After startup, the S1Waccomplished a simulated voya and at full power most of the way during the 96 hour the feasibility of atomic ship propulsion long before <i>U</i> advanced design equipment and operated as part of the	SS Nautilus set out to sea. Later, the S1W tested	,		

The SNAPTRAN program extended the SPERT reactor safety testing program to aerospace applications. Three test series, involving three reactors, investigated the behavior of SNAP10A/2 fuel under large-transient,

Name	Acronym	Reactor Startup	Last Day of Operation
Systems for Nuclear Auxiliary Power (SNAP) 10A Transient No. 1 (continued) power-excursion conditions. SNAPTRAN-1, located at Test Area North, was subjected to non-destructive tests in conditions approaching but not resulting in damage to the zirconium-hydride-uranium fuel.			
47. Systems for Nuclear Auxiliary Power (SNAP) 10A Transient No. 3 SNAPTRAN-3 was the first of two destructive tests on a version of the small space reactor (SNAP10A/2) designed to supply auxiliary power in space. The test, conducted at TAN's Initial Engineering Test Facility of April 1, 1964, simulated the accidental fall of a reactor into water or wet earth such as could occur during assembly, transport, or a launch abort. The test demonstrated that the reactor would destroy itself immediate instead of building up a high inventory of radioactive fission products.		4-1-64	4-1-64
48. Systems for Nuclear Auxiliary Power (SNAP) 10A Transient No. 2 This test version of the small space reactor, SNAP 10A/2, was intentionally destroyed on January 11, 1966. It provided information on the dynamic response, fuel behavior, and inherent shutdown mechanisms of these reactors in an open air environment. In normal operation, the control drums of the SNAP10A/2 were rotated to obtain criticality after the reactor had been placed in orbit. In case of a launch abort, however, impact on earth might cause the drums to rotate inward, go critical, conceivably destroy itself, and release fission product to the surrounding environment. The test data contributed to an understanding of reactor disassembly upon impact and methods for assessing or predicting the radiological consequences.	he	1965	January 11, 1966
49. Thermal Reactor Idaho Test Station THRITS was a low-power reactor located at TAN. Its nuclear core was arranged in two halves of a vertical, aluminum, honeycomb-like matrix. The reactor could not be operated until the two halves were brought together to form the critical fuel mass. Operators mocked up reactor design concepts for thermal and fast-neutron reactor systems to obtain basic physics and design data for such concepts.	THRITS Split-Table Reactor		1964
Part of the safety program for fast breeder reactors, TREAT was a uranium-oxide-fueled, graphite-moderated air-cooled reactor designed to produce short, controlled bursts of nuclear energy. Located at Argonne-West, its purpose was to simulate accident conditions leading to fuel damage, including melting or even vaporizati of test specimens, while leaving the reactor's "driver" fuel undamaged. Early studies determined the effect of extreme energy pulses on prototype fuel pins designed for EBR-II. TREAT tests provided data on fuel-cladding damage, fuel motion, coolant-channel blockages, molten-fuel/coolant interactions, and potential explosive forces during an accident. The data helped refine computer simulations of reactor accidents, and, ultimately, design reactors with greater inherent safety.	on	2-23-59	April 1994
51. Zero Power Physics Reactor (Earlier name - Zero Power Plutonium Reactor) ZPPR, a low-power critical facility located at Argonne-West, provided reactor physics data for any type of fast neutron spectrum reactor, from tiny space-power reactors to large commercial breeder reactors. The (full-size) reactor core configuration to be studied was mocked up in two halves, the fuel loaded into a honeycomb lattice in each of the separated halves. Extrapolation from the zero-power measurements to full-power conditions was readily achievable. Upon moving the two lattice together, ZPPR was brought to a low power, critical state by control rods. Heat removal was by air flow over the fuel elements.	ZPPR	4-18-69	April 1992 Standby
52. Zero Power Reactor No. 3 This was a low-power split-table reactor that achieved criticality by bringing two halves of a fuel configuration together. Alow-power reactor, ZPR-III was used to determine the accuracy of predicted critical mass geometries and critical measurements in connection with various loadings for makeup of fast-reactor core designs. The cores of EBR-II, Fermi, Rapsodie, and SEFOR reactors were originally mocked up in this facility.	ZPR-III	October 19	955 November 1970
Experimental critical assembly results in this field were almost completely lacking before this reactor started up. The reactor was placed on standby in 1970 and later went on display in the EBR-I Visitor Center.).		

APPENDIX (

Processing Runs, Idaho Chemical Processing Plant

Detween 1953 and 1988 the Chem Plant recovered from spent reactor fuel 31,432 kilograms of uranium containing uranium-235. At times, the plant also recovered radioactive lanthanum (RaLa), neptunium, and radioactive krypton and xenon for use by private industry or other nuclear facilities.

Among the more frequent sources of fuel were the MTR, ETR, ATR, and STR (S1W), ZPR-III, EBR-I, and EBR-II—all NRTS/INEL/INEEL reactors. In addition, the plant processed SL-1 fuel, SNAPTRAN debris, and fuel from OMRE, BORAX, and SPERT reactors, and other test materials. The largest single source of reprocessed fuel was from naval nuclear propulsion reactors: prototype reactors, submarines, cruisers, and other vessels

Other sources included SRP (Savannah River Plant), Hanford, LITR (Low Intensity Test Reactor, Oak Ridge), SIR (Submarine Intermediate Reactor), OWR (Omega West Reactor, Los Alamos), GETR (General Electric Test Reactor), LPTR (Livermore Pool-type Reactor), BGRR (Brookhaven Graphite Research Reactor), ASTR (Aerospace Systems Test Reactor), JRR (Japanese Research Reactor), NRX (Nuclear Engine Reactor Experiment, Jackass Flats, Nevada), SER (Sandia Engineering Reactor), KUR (Kyoto University Reactor), STIR (Shielding Tests Irradiation Reactor), ORR (Oak Ridge Research Reactor), JANUS (Biological Research Reactor, Argonne National Laboratory), BMI Reactor (Battelle Memorial Institute of Columbus, Ohio) ML-1 (Mobile Low-Power Reactor), SFR (Segmented Fast Reactor), and many other university and research reactors.

Run

Number	Fuel	Process Period
1	Hanford C and J slugs	2-53 to 8-53
2	MTR, LITR, NRX, ORNLshielding Cold test EBR-I Core 1	10-53 to 12-53 7-54 to 7-54
3	EBR-I Core 1, NPCold Run, MTR, LITR, BORAX, BORAX scrap, Hanford C and J slugs	7-54 to 2-55

Run Number	Fuel	Process Period
4	Hanford J slugs, MTR, BORAX bulk shielding,	
	BORAX, LITR	3-55 to 7-55
Cold tests	Cold test SRPreject slugs	9-55 to 11-55
5	Hanford J and C slugs, Chem Dev Test SRP	10.55 . 0.56
	reject slugs	12-55 to 3-56
6	MTR, LITR, BORAX, CP-3, CR	3-56 to 5-56
7	Hanford C and J slugs, CR, MTR, BORAX,	
	LITR, ANL plates, LM slugs, STR, RaLa MTR	5-56 to 3-57
	CPM cold start with LM slugs	8-57 to 9-57
8	LM slugs, RaLa MTR	10-57 to 12-57
9	STR	12-57 to 1-58
10	Hanford C slugs, RaLa MTR, SRP LM slugs	1-58 to 2-58
11	SRP LM slugs, SRP Tube, MTR, RaLa MTR,	
	Chalk River, SRP Tube	5-58 to 11-58
12	SRP slug, SRP Tube, NRX, RaLa MTR	12-58 to 4-59
13	SRP Tube, SRPslug, SRP Tube ends, Chalk River	4-59 to 8-59
14	SIR, OMRE, BMI, RaLa MTR	7-59 to 12-59
15	MTR, RaLa MTR, ETR, LITR, Convair (ASTR),	
	Hanford C, J, and KW slugs, SRPLM slugs	12-59 to 2-60
16	SIR, RaLa MTR	2-60 to 3-60
17	STR, RaLa MTR	3-60 to 4-60
18	ETR	1-61 to 2-61
19	MTR, ETR, BORAX IV, RaLa MTR, Hanford C and J slugs, LITR, Chalk River, CP-5, LPTR, GTR (Convair),	
	OWR, SL-1 scrap	12-61 to 2-62
	SL-1	10-62 to 10-62
20	MTR, ETR, RaLa MTR, SPERT, GETR, BRR, SL-1,	
	BNL, LITR, CP-5, LPTR, GTR (Convair), OWR, WTR,	
	BORAX III, SUSIE, Hanford AEC, Hanford Rey, NRU	6-63 to 9-63

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Run Number	Fuel	Process Period
21	BGRR, NRX, McMasters, NRU, NRL, SWE, IRL,	
	University of Michigan—FNR, GTR, MTR, OWR,	
	LPTR, LITR, UF, ETR, CP-5, STR, SPERT, NASA	6-64 to 12-64
	Cold Zr, unirradiated Zr scrap, PWR Core I Seed I,	
	Zr, EBR-I Core 3 codissolution, EBR-I Core 3,	
	SNAPTRAN 2/10A-3 core debris	1-65
22	VBWR, Atomics International UO ₂ SO ₄	4-65 to 6-65
23	Cold from ATR, MTR, ETR, and SPERT; MTR, ETR,	
	LITR, LPTR, OWR, SPERT, GTR, ASTR, GETR,	
	EBR-II Vycor glass, EBR-I Mark 2, plastic-coated	
	Al fuel plates	12-65 to 1-66
24	JRR-2 Core 1, NRX, NRU, BGRR, EBR-II Vycor glass,	
	JRR-2 Cores 2 and 3	3-67 to 9-67
25	MTR, WSU, ETR, LITR, LPTR, OWR, GTR, CP-5,	
	SER, IRL, GETR, NRL, graphite leaching, Zr, EBR-II	
	Vycor glass	4-68 to 6-68
26	Zr, MTR, ETR, GETR, Korean, SER, LITR, AFNETR,	
	JRR-2, KUR, LPTR, OWR, ATR, SPERT, ZPR-III,	
	SNAPTRAN 2/10-2 debris	8-69 to 10-69
	ETR types	1-70 to 4-70
27	Zr, JRR-2 (6 batches), EBR-II scrap, WADCO	2-71 to 7-71
28	Zr, ETR, custom miscellaneous	6-72 to 9-72
29	EBR-II, EBR-II slurry and denitrator product	1-73 to 5-73
30	Zr, GETR, ATR, MTR, MTR 20%, TRAscrap, JRR, ETR,	
	CP-5, OWR, JMTR, Juggernaut, KUR, UM, SER, LPTR,	
	EBR-II Vycor glass, G.G.A. Thermionic, ETRC plates,	
	University of Wyoming UO ₂ SO ₄ , Atomics International fission	
	disc, HTRE scrap, Walter Reed Army Hospital, Nuclear	
	Test Gauge/Split Table Reactor, HTGR secondary burner	
	ash leaching, BMLfission disc	2-74 to 5-74
31	EBR-II, APPR cold fuel scrap	2-75 to 5-75
32	Zr, PWR	5-76 to 9-76
33	Godiva reactor fuel, HTRE, ATR, MTR, LPT, ETR,	
	GETR	3-77 to 6-77

Fuel	Process Period
EBR-II, OMRE, SPERT, ORNL-17-1, BMI,	
Kinglet, Sandia (Godiva reactors), PBF metallurgical samples	8-77 to 9-77
Zr, custom	7-78 to 3-79
Zr, Rocky Flats U ₃ O ₈ , GETR, OWR, STIR,	
LPTR, UCLA-MTR, ATR, ETR, ATR-XA	9-80 to 3-81
EBR-II, Los Alamos metal fuel scrap, Rocky Flats	
U_3O_8 , Rover cold	8-81 to 11-81
ETR, BSR, ATR, OWR, ORR, HFR-PETTEN,	
${\rm SAPHIR,GETR,FRG,FRJ/FRM,SFR,UO_{\tiny 2}SO_{\tiny 4}}$	9-82 to 11-82
Rover, Sandia, Rocky Flats, cold FLUORINEL,	
FLUORINEL Phase 1 cold run	4-83 to 6-84
ITAL, FRG, DR-3, UCLA, MURR, OWR, HFBR,	
LPTR, TR-1, ATR, BSR, ORR, HMI, Triton, FRJ-2,	
HRF, BR-2, ORPHEE, ASTRA, SRF, R-2, JUNTA,	
McMaster, JRR-2, JMTR, JANUS, SR, UCSB	
UO ₂ SO ₄ , FLUORINELPhase II cold run,	
FLUORINEL pilot plant	8-85 to 1-86
FLUORINEL	10-86 to 10-87
FLUORINEL, EBR-II Vycor glass, BYU UO ₂ SO ₄ ,	
EBR-II fuel scrap, ANL-E fuel scrap	12-87 to 7-88
	EBR-II, OMRE, SPERT, ORNL-17-1, BMI, Kinglet, Sandia (Godiva reactors), PBF metallurgical samples Zr, custom Zr, Rocky Flats U ₃ O ₈ , GETR, OWR, STIR, LPTR, UCLA-MTR, ATR, ETR, ATR-XA EBR-II, Los Alamos metal fuel scrap, Rocky Flats U ₃ O ₈ , Rover cold ETR, BSR, ATR, OWR, ORR, HFR-PETTEN, SAPHIR, GETR, FRG, FRJ/FRM, SFR, UO ₂ SO ₄ Rover, Sandia, Rocky Flats, cold FLUORINEL, FLUORINEL Phase 1 cold run ITAL, FRG, DR-3, UCLA, MURR, OWR, HFBR, LPTR, TR-1, ATR, BSR, ORR, HMI, Triton, FRJ-2, HRF, BR-2, ORPHEE, ASTRA, SRF, R-2, JUNTA, McMaster, JRR-2, JMTR, JANUS, SR, UCSB UO ₂ SO ₄ , FLUORINELPhase II cold run, FLUORINEL pilot plant FLUORINEL FLUORINEL, EBR-II Vycor glass, BYU UO ₂ SO ₄ ,

Sources:

M.D. Staiger, *Calcine Waste Storage at the Idaho Nuclear Technology and Engineering Center*, Report No. INEEL/EXT-98-00455 (Idaho Falls: Lockheed Martin, June 1999), Appendix pp. A-163 to A-168.

Dieter Knecht, et al., "Historical Fuel Reprocessing and HLW Management in Idaho," *Radwaste Magazine* (May 1997).

Leroy Lewis, Science and Engineering Fellow, Bechtel BWXT Idaho, contributed corrections to the lists published in the above two documents.

APPENDIX D

Criticality Accidents, Idaho Chemical Processing Plant

criticality accident is an unintended amassing of a fissionable material (like uranium) which results in the fissioning of the material in a chain reaction. In such an event, fission products such as heat, gamma radiation, neutrons, gases, and other emissions are released by the nuclear reaction.

The designers of chemical processing, fuel fabrication, and other plants that handle fissionable material employ a variety of strategies to avoid accidental amassing of enough material to initiate a chain reaction. The examples below refer to uranium, but similar principles would apply to the management of any other fissionable material:

- Geometric control: the dimensions of the containers and conveyors of uranium
 make it impossible to reach a critical mass. At the Chem Plant, for example,
 certain dissolver vessels and storage vessels were no more than five inches in
 diameter. Spacing of vessels was also important, with two feet between vessels
 required to prevent a criticality.
- Concentration control: Where chemical processes involve evaporation or precipitation reactions which could result in the concentration of the uranium, vessels and containers are sized to prevent accidental accumulations of a critical
 mass. Appropriate dilution may also be used to keep the solution concentration
 below a minimum value to prevent criticalities.
- Mass control: In handling enriched uranium, the quantity that can be handled at any one time is limited to a specified, known-to-be-safe number of grams of material.
- Administrative control: Operational procedures may require two or more people to approve if a particular procedure could lead to a loss of control. Checkoff points, guide limits, process alarm systems, color-coding of certain valve handles, key-only procedures, personnel training, and other controls accompany non-routine and many routine procedures.

1. Criticality Accident of October 16, 1959

A bank of storage cylinders containing a uranium solution was air-sparged (air was bubbled violently into the solution to mix it). The cylinders were geometrically safe, but the sparging initiated a siphon that transferred 200 liters of the solution to a 5,000-gallon tank containing about 600 liters of water. The resulting criticality lasted about twenty minutes.

No workers were exposed to gamma or neutron radiation, as the criticality occurred in a cell below ground when no one was in the vicinity. Airborne activity spread through the plant through vent lines and drain connections, triggering alarms and an evacuation. Two people who evacuated received significant beta doses (with no detectable medical consequences) as they passed areas where radioactive gas was being released into the room from floor drains.

The incident resulted in the placement of new valves, restrictions on air-flow lines when sparging, installation of water traps, and other measures before the plant restarted.

2. Criticality Accident of January 25, 1961

About 40 liters of uranyl nitrate solution (200 grams of uranium per liter) was forced upward from a 5-inch-diameter section of an evaporator into a 24-inch-diameter vapor disengagement cylinder, well above normal solution level. Analysts later assumed that air entered associated lines while operators were attempting to clear a plugged line and improve a pump. When the air bubble reached the evaporator, solution was expelled from the lower section, and a momentary criticality occurred in the upper section. Radiation triggered alarms, but no personnel received more than 100 mrem exposure. Concrete shielding walls surrounded the location of the criticality; the vent system prevented airborne activity from entering work areas; and equipment design prevented a persistent excursion. No equipment was damaged.

Management thereafter restricted the use of air pressure to move liquids and clear lines. A borated steel grid was installed in the disengagement cylinder. Boron is a nuclear "poison" that absorbs neutrons, helping prevent criticalities.

3. Criticality Accident of October 17, 1978

During the first solvent extraction cycle in the recovery of uranium from spent fuel, the uranium was extracted from the dissolution solution and then scrubbed, stripped, and washed in various process columns to separate the uranium from fission products. The criticality occurred in the scrub column (a long narrow, vertical tank). Water had leaked into the tank where the scrub chemical, aluminum nitrate, had been made up, and reduced the aluminum nitrate concentration. But this was

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not known to operators because an alarm had not been repaired and was inoperable. Also, operators had not sampled the scrub solution to determine whether the aluminum nitrate was above the required concentration. As a result, the solution was too dilute to force the uranium into the organic phase and instead extracted small amounts of it out on the organic phase into the aqueous phase and accumulated it into the large-diameter disengaging head.

Over a period of a month, uranium continued to accumulate until it reached a concentration in the aqueous phase high enough to achieve criticality. It accumulated in a large-diameter part of the column designed to separate the organic phase from the aqueous phase. The criticality reaction continued for about a half an hour before the operators responded to the slight pressure build-up and took steps to terminate the reaction.

The criticality occurred in a well-shielded location inside a process cell and resulted in insignificant radiation exposures to personnel or damage to equipment.

The operation and management failures associated with this criticality led to a significant reassessment and evaluation of Chem Plant operations. A Plant Protection System (which consisted of a variety of changes in procedures, operating limits, sampling protocols, specifications, warning systems regarding analytical samples, and others) was installed to preclude this type of accident from happening again.

APPENDIX E

R&D 100 Awards

Cryogenic ZAWCAD • 1999

Research Team: Dennis Bingham, Russell Ferguson, Gary Palmer, Douglas Stacey, Richard Swainston, Carl A. Dunn, Gerald Decker

Description: Cryogenic ZAWCAD is a remarkable, patented cutting and cleaning tool that will make many industrial processes safer and more environmentally friendly by performing hazardous and non-hazardous cleaning and cutting operations while minimizing secondary waste. This unique system uses as its cutting/cleaning medium a harmless atmospheric gas that dissipates after use. As a result, there is no secondary wastestream and no cross contamination. Yet, it can cut with the precision of the most advanced cutting tool and clean or abrade surfaces with finer control, more aggressiveness, and greater efficiency than other cleaning technologies-all without creating a secondary waste to clean-up, dispose of or treat.

Cryogenic ZAWCAD is a highly controllable technology, adjustable for temperature, speed, and aggressiveness.

Tractrix Valve • 1999

Research Team: John Wordin, Pio Park

Description: The Tractrix Valve is a revolutionary, self-sealing valve that doesn't leak and doesn't wear out. Based on a geometric shape called a "tractrix curve," this innovative, plugtype valve wears less than competing plug and ball valves and seals tighter the more it wears—essentially "wearing in" each time it is opened and closed. It requires up to 90% less torque to actuate and can be made of any construction material for the most severe or simple applications. This cost-competitive new valve represents a leap in technology that could make common plug and ball valves obsolete for many applications—particularly those that are environmentally sensitive.

High Void-Fraction Multiphase Flowmeter • 1999

Research Team: James Fincke, Darrell Kruse, Daniel J. Householder, Bulent Turan, Doyle Gould, Charles Ronnenkamp

Description: The INEELHigh Void-Fraction Multiphase Flowmeter solves one of the natural gas industry's most difficult measurement problems: cost-effective measurement of "wet gas" (mixed-phase flows of 5% liquid by volume). This innovative flowmeter represents a major economic breakthrough that can impact over 300,000 natural gas wells in the U.S. alone. It offers extraordinary size and cost savings over existing technology, and is the only device to provide real-time wet-gas measurement at the wellhead. It brings better fiscal management to producers, maximizing gas recovery. Its compact size will simplify facilities design, and its low cost will reduce capital investment and total gas production costs. Most importantly, the INEEL Flowmeter gives producers a reliable, economical means of managing natural gas reservoirs for the first time, conserving a precious natural resource.

Maverick Tank Inspection Robot • 1999

Research Team: Thor Zollinger, Kerry Klingler, Charles B. Isom, Kerry Trahan, Scott Bauer, Don Hartsell

Description: The Maverick is a submersible, robot-based system that offers safe, practical, and cost-effective inspection of in-service, aboveground storage tanks (AST). This patented technology provides direct and indirect cost savings of 75% or greater over traditional manual inspection methods, reduces the inspection process from four weeks (or more) to a few days, eliminates worker exposure to hazardous conditions, and enables tank owners to continue using their tanks during inspections, saving them tens of thousands of dollars in previously "lost" revenue. Maverick

also is the only robotic inspection system certified for use in hazardous and potentially explosive fuel environments (Class I, Division 1, Group D) such as gasoline and other fuel oils.

The robot's payload includes a multi-channel ultrasonic sensor system to map and correlate metal thickness data, an onboard video system to provide a detailed view of the tank bottom, and position-tracking sensors so technicians know the exact location of the Maverick, and any problem spots, at all times.

Supercritical Fluid Slashing System • 1999

Research Team: Mark Argyle and Alan Propp

Description: Before threads can be woven into fabric, they must be "sized," a process that adds a strengthening and smoothing coating to the thread. The INEELSupercritical Fluid Slashing System (SFSS) is a cheaper, faster, smaller, and more environmentally correct method for coating threads with size, one that replaces centuries-old technology. The INEELmethod transports the size (starch or polyvinyl alcohol) in a very high-pressure "supercritical fluid" that has properties of both a fluid and a gas. Individual threads pass through pressure gradient tubes, where the supercritical sizing mixture is forced into the threads. The efficient method reduces the amount of water, starch, and polyvinyl alcohol that textile manufacturers dispose of. The SFSS specifically addresses an industry "wish list" to provide uniform coating, reduce yarn hairiness, reduce the amount of sizing material, eliminate standing baths, and minimize drying. Because it significantly increases sizing efficiency, the SFSS can double production throughput for improved profitability.

Electro-Optic High-Voltage Sensor • 1998

Research Team: Thomas M. Crawford, James R. Davidson, Gary D. Seifert

Description: The Electro-optic High-voltage Sensor (EHVS) is a safe, small, non-electrical optical sensor that uses photons instead of electrons to measure high voltages on power lines. The most unique aspect of this technology is that the sensor does not have to be in electrical contact to effect a measurement, but simply within the conductor's electronic field, a key advantage over the large transformers conventionally used for voltage measurement at power distribution sites. The EHVS device offers substantial improvement over potential transformers in cost, ease of installation, range of response to voltage fluctuations, and richness of applications.

Rapid Solidification Process Tooling • 1998

Research Team: Kevin McHugh

Description: Rapid Solidification Process (RSP) Tooling technology is a fast, low-cost alternative to conventional fabrication of precision tooling used in the manufacture of nearly all mass produced products, from cell phones to automobiles. This new, molten metal spray-forming technology promises to reduce the cost and lead time for producing tooling by a factor of 5 to 10, substantially shortening the time it takes industry to get products to market. Unlike other alternative tooling approaches, RSPTooling technology makes it possible to create tooling from hard tool steels at the rate of 2,000 lb/hr or more, suitable for the largest auto industry tooling requirements.

Malt-Based Antimicrobial • 1998

Research Team: Karen B. Barrett

Description: The Malt-Based Antimicrobial is a naturally occurring biopesticide derived from malted cereal grains. Developed as an environmentally sound solution to agricultural crop protection, this product represents a major breakthrough in pesticide research. It offers an extraordinary taxonomic range—unrivaled by chemical fungicides, is easily and inexpensively produced, has an excellent shelf life; and can be used to protect crops in the field, in storage, and during transport. Most importantly, this new

biopesticide is harmless to people, animals, and the environment; and is derived from a plentiful renewable resource: common cereal grain.

Nanocrystalline Composite Coercive Magnet Powder • 1997

Research Team: J.D. Branagan, J.A. Hyde, C.H. Sellers, K.W. Dennis, M.J. Kramer, R. W. McCallum

Joint entry: Ames Laboratory, Dr. R. William McCalum

Description: The development and application of a new alloying approach in rare earth-based permanent magnet systems resulted in the development of advanced alloys with a nanocrystalline composite microstructure. Atomization processing of these new alloys resulted in significant improvements in hard magnetic properties and processability over previous alloys, allowing the possibility of near term, high volume, low cost production of atomization materials.

Advanced Tensiometer • 1997

Research Team: Joel M. Hubbell, James B. Sission

Description: The Advanced Tensiometer is an instrument that measures how tightly water is held to soil in the unsaturated zone, a region that extends from the earth's surface to the aquifer. The Advanced Tensiometer's breakthrough design helps investigators determine the direction and rate of water movement at depths and with accuracies not possible before, ushering in a new era for monitoring waste disposal sites, safeguarding drinking water supplies, and controlling agricultural irrigation systems.

Gamma Neutron Assay System • 1995

Research Team: R. Aryaeinejad, J.D. Cole, R.C. Greenwood

Description: The Gamma Neutron Assay System (GNAT) is a new, nonintrusive, and unique patented technique of identifying fissile materials and their isotopic ratios in bulk quantities and in a field environment. It resolved, for the first time, problems of assaying and tracking special nuclear material (SNM) not previously possible, which are important in arms control, nonproliferation, and nuclear weapons dismantlement.

Biocube Aerobic Biofilter—A Biofilter for Treatment of Toxic Gases and Vapors • 1993

Research Team: W.A. Apel, F.S. Colwell, A.S. Espinosa, E.G. Johnson, B.D. Lee, M.R. Wiebe, W.D. Kant, P. Melick, B. Singleton

Joint entry: EG&G Roston, W.D. Kant

Description: The BiocubeTM Aerobic Biofilter is a landmark product that ushers in a new era for degradation of toxic vapors and gases. It is novel, effective, and economical vs. conventional technologies, and is the first modular and mobile biofilter.

Portable Isotopic-Neutron Source Chemical Assay System • 1992

Research Team: A.J. Caffrey, J.D. Cole, L. Forman, R.J. Gehrke, R.C. Greenwood, K.M. Krebs, M.H. Putnam

Description: The Portable Isotopic-neutron source (PINS)-based non-destructive assay system distinguishes chemical weapons (e.g., nerve gas) from high-explosive munitions for treaty verification.

Pulsed Extraction Secondary Ion Mass Spectrometer • 1992

Research Team: D. Applehans, D.A. Dahl, J.E. Delmore

Description: This is a new type of Secondary Ion Mass Spectrometer using a patented secondary ion Pulsed Extraction technique that prevents sample charging and allows the positive and negative ion spectra to be collected simultaneously, making possible analyses that previously could not be performed.

Sulfur Poisoning Resistant and Regenerable Hydrogenation Catalysts • 1990

Research Team: Randy B. Wright

Description: This entry is a new, unique, and advanced method for the preparation of highly active, sulfur poisoning resistant and repeatedly regenerable hydrogenation catalysts. This method utilizes the controlled manipulation of chemically induced surface segregation processes in conjunction with intermetallic compounds and binary alloys to design and synthesize a wide range of

APPENDIX E

active catalysts. As specially applied to nickelbased intermetallic compounds, this approach provides a technique by which highly active hydrogenation catalysts can be prepared and regenerated by elevated temperature oxidation/ reduction treatment of the starting material.

FiberOptic Moire Interferometry System • 1990

Research Team: V. Deason, M.B. Ward

Description: The FiberOptic Moire Interferometry System, Model FMI 1700, is a major advance in an important new techniquediffraction moire interferometry. Diffraction moire is used for the study and measurement of distortion, stress, and fracture. The FMI was developed at the INELin response to serious failings to the existing diffraction moire systems. The FMI utilizes advanced optical fiber components in a compact portable unit to replace an optical table full of standard optical devices. These fiber optic components are in all cases smaller, lighter, and more stable than the discrete components normally used. At the same time, the FMI greatly simplifies the experimental process, which until now has been complicated and tedious using conventional equipment.

Diffraction Moire Interferometry, for which the FMI was developed, allows the researcher to make highly accurate (better than one Micron resolution) measurements of deformation. This deformation could be the result of stresses on a ship's hull, an airplane's frame, bridge, piping in a nuclear plant, or other critical area. Understanding the relationship between stress, deformation, and such factors as aging, load, corrosion, and material properties is crucial to reducing the heavy burden on the economy and society of structural failures (estimated at tens to hundreds of billions of dollars annually, plus extensive loss of life and productivity).

Diffraction moire measures deformation in a specimen by creating full two-dimensional maps of the corresponding deformation in a diffraction grating bonded or marked on the surface of the specimen.

Finnigan MAK Gas Mass Spectrometer Model 271/251 • 1990

Research Team: R. Rankin, K.W. Guardapee, L.L. Dickerson

Joint entry: Finnigan MAT Corporation

Description: The 271/251 gas mass spectrometer defined the state-of-the-art in instrumentation for the analysis of noble gases in the environment. The unique aspect of this instrument is that it combines two diverse gas analysis functions, usually requiring separate instruments, into a single entity. The instrument performs both gas composition and gas isotope ratio analyses. In addition, typical isotopic precision capabilities with gas mass spectrometers were on the order of 0.03%. This product has been able to exceed these values by more than an order of magnitude (0.001%), thereby significantly extending the state-of-the-art in high precision gas isotopic analysis. Highly precise measurement of the concentrations of the gas isotopes in the atmosphere is indispensable to environmental studies involving nuclear facilities. This work has been recognized internationally; instruments based on these designs are now in use in Europe and Asia.

Simion PC/PS2 4.0 • 1989

Research Team: D.A. Dahl, J.E. Delmore, A.D. Applehans

Description: SIMION PC/PS2 4.0 is a personal computer program for designing and analyzing charged particle (ions and electrons) lenses, ion transport systems, and all types of mass spectrometers and surface probes that utilize charged particles. The program, which became available in June, 1988, has exclusive capabilities that significantly expand the number and types of problems that can be addressed, problems that heretofore were impossible to model with existing programs.

Neutral Molecular Beam Surface Probe • 1988

Research Team: J.E. Delmore, A.D. Apprelhans, and D.A. Dahl

Description: This device produces a well focused beam of high energy neutral sulfur hexafluoride molecules at energies ranging from 3 to 23 keV that can be transported many meters under vacuum while retaining sharp focusing, to probe the

few molecular layers of a sample's surface. The primary function of the Neutral Molecular Beam Surface Probe is the analysis of surfaces of nonelectrical-conducting materials. Asimilar technique has been used with charged atomic particle beams for many years to analyze surfaces of electrically conducting materials, although the technique is applied with great difficulty to insulating materials. Other techniques, notably fast neutral atom beams (FAB sources) have been used with some success, but the nature of their production precludes sharp focusing. In addition, the FAB source must be mounted quite close to the specimen, and that constrains the secondary ion source design. The new Neutral Beam is much easier to use, allows much sharper focus, and increases sensitivity about 1000 fold over systems using charged particle beams on insulating specimens.

Biodegradation System for Toxic Organic Waste Processing • 1988

Research Team: J.H. Wolfram, R.D. Rogers

Description: Disposing of hazardous waste is a high-priority item for every organization that produces it. Virtually every hospital, research university, and biotechnology company in the U.S. produces small quantities of hazardous wastes of organic compounds (e.g., toluene, xylene, and pseudocumene). The first two of these compounds are on the EPApriority pollutant list. A mixture of these compounds is know as liquid scintillation cocktail when the mixture also contains radioactive materials. The Biodegradation System presented here introduces organisms to the cocktail that detoxify it at very high levels of efficiency. The system includes a bioreactor (where living cells "feed" on the toxic materials and produce carbon dioxide), accessory hardware, and a supply of the detoxifying microorganisms. Once installed, the system will continually bioprocess an inflow of the cocktail, rendering it safe for conventional disposal through the sewer system if the radioactivity is within prescribed limits, or as low-level radioactive waste otherwise. This eliminates the high cost of packing and transporting the mixed waste to the very few authorized disposal sites. Sanctioned methods for handling the waste at present are incineration and longterm storage. Converting the waste at the site eliminates dependence on these methods and the cost associated with them.

Improved Iron-Based Alloys from Noble Gas Doping • 1988

Research Team: John E. Flinn

Description: The primary function of this product is to strengthen alloys for high-temperature applications. Noble gas atoms (e.g., those of helium or argon), when entrapped during the processing of iron-base powders, stabilize the microstructure and strengthen the alloy produced. The alloying addition forms numerous small and very stable clusters with vacancies (missing atom sites) during rapid cooling. The formation of the clusters is due to the high binding energy between the noble gas atoms and the vacancies that are created by high temperature exposure (i.e., heat treating). Because they retard microstructure coarsening, the clusters allow fine microstructures to be retained during exposure to high temperatures. Cluster presence provides a form of solid solution and dispersion strengthening. The strengthening is further enhanced during aging heat treatments because the clusters provide nucleation, or preferred, sites for the formation of precipitates such as carbides. The fine dispersion of a large number of precipitates significantly improves the strength of the alloy. This process is applicable to all ironbase alloys, particularly to stainless steels.

Oxynitride Braze Method for Joining Silicon Nitride Ceramics • 1988

Research Team: R.M. Neilson, D.N. Coon, S.T. Scheutz, R.L. Tallman

Description: Structural ceramics are potential substitutes for strategic and/or critical materials. However, many potential structural ceramic applications require components that are too intricate or too large to be fabricated with existing techniques. The invention is a method for joining silicon nitride ceramics to produce large parts and/or parts that have complex geometries in which the high-temperature mechanical properties of the joined part are comparable to that of the original ceramic components. For certain applications (e.g., aerospace, engines, chemical processing) ceramic parts, and complex shapes are preferred to metals. Reasons include: increased service operating temperatures, greater strength and increased corrosion resistance at the higher temperatures, greater thermodynamic efficiency in energy conversion devices, lower density and,

therefore, lower inertia, lower cost of raw materials, and conservation of possibly strategic and/or critical materials in some applications. Structural ceramics are, however, difficult to form in either larger size parts or in complex geometries because the forming process usually requires some combination of high temperature and high pressure. Some method of joining the ceramics while maintaining the structural integrity and thermodynamic qualities of the ceramic is needed to produce the larger and/or more complex shapes required in many applications. The oxynitride braze method for joining silicon nitride ceramics presented here has been demonstrated to be an effective joining technique using either a hot isostatic press or a graphite resistance furnace with small nitrogen overpressure. In this process, oxynitride glass brazes are used to join silicon nitride ceramics. The glass uses are comparable in composition to the grain boundary phase present in the ceramic pieces that results from the densification process used to consolidate the silicon nitride powders.

Die-Target for Dynamic Consolidation of Powders • 1987

Research Team: John E. Flinn, Gary E. Korth

Description: Die Target for Dynamic Consolidation of Powders is a new and improved method of consolidating metal monoliths from rapidly solidified powders (RSP). The Die-Target controls dynamic stress waves produced by detonation of explosive charges to consolidate RSP alloys. With each detonation, the Die-Target produced four fully consolidated, fully dense, crackfree monoliths for test specimens. This process does not produce high generalized temperatures in the powders, which could seriously alter the microstructure and desirable properties for the RSP. Monoliths produced by the dynamic consolidation of RSPalloys (e.g., stainless steel) have improved mechanical properties, improved corrosion resistance, chemical homogeneity, extended solubility limits, very fine microstructures, and desirable metastable phases. At present, the primary use of the Die-Target is as a research tool. However, the theory and principles underlying the design hold promise for industrial and commercial applications of the DIE-Target where advanced materials that are harder and stronger are needed.

Vision System for High Luminosity Processes • 1986

Research Team: Jon Bolstad, M.B. Ward, C.L. Shull

Description: This system produces high-quality video imagery of industrial or experimental processes which are normally obscured by high luminosity of an electric arc, a plasma, or a combustion flame. It has particular application in electric arc welding where detailed vision of the welding pool, electrode, and liquid/solid interface is required. The welding site is illuminated by pulsed laser light transported to the welding torch by one or more optical fibers. The sensor assembly incorporates objective optics, a laser line filter, a microchannel plate image intensifier tube, and a CCD video camera. The intensifier tube is shuttered electronically in synchronism with the flash from the laser source, which occurs only once per video frame (or some multiple thereof). The shuttering interval (about 100 nanoseconds) is very small in comparison with the 33 millisecond integration time of a standard video camera. The welding arc light is almost totally eliminated from the video picture. Visibility through the arc is regained, and extreme variation in brightness across the picture is removed. The video imagery is much superior to standard video for interpretation by eye and by electronic image processing equipment.

Notes

ABBREVIATIONS

IHS Idaho Historical Society

Bou Boise State University Library

Special Collections

HREX Human Radiation Experiments,

Internet site:

http://tis.eh.doe.gov/ohre/

CHAPTER ONE

Epigraph: Susan M. Stacy, *Conversations, A Companion Book to Idaho Public Television's Proceeding On Through a Beautiful Country* (Boise: Idaho Public Television, 1990), 47.

- Mike Atwood, interview with author, January 22, 1999, recounted his story as retold in this chapter.
- 2. "Fourteen Stockmen Testify at AEC Land Hearing," *Post-Register*, May 3, 1950, 2.
- See Dr. Bill Hackett, et al, Geohydrologic Story of the Eastern Snake River Plain and the Idaho National Engineering Laboratory (Idaho Falls: Idaho Operations Office, 1986), 11.
- (Lava tubes) Hackett, Snake River Plain, 11;
 (date) personal communication from Dr.
 Richard P. Smith to Julie Braun, INEEL,
 March 9, 1999. Lava flows at Craters of the
 Moon National Monument west of the
 INEELmay be only 2,000 years old.
- 5. Mike Atwood, January 22, 1999.
- 6. E.S. Lohse, "Aviator's Cave," *Idaho Archeologist* 12 (Fall 1989): 23.
- Susanne J. Miller, *Idaho National Engineering Laboratory Management Plan for Cultural Resources (Final Draft)* (Idaho Falls: Lockheed Idaho Technologies
 Company Report DOE/ID-10361, Revision 1, 1995), 2-17.
- 8. Lohse, "Aviator's Cave," 23-28.
- 9. Miller, *Cultural Resources Plan*, pp. 2-18 to 2-21.

- (Homesteading) Miller, Cultural Resources Plan, pp. 2-21 to 2-22; (quotation) Hugh Lovin, "Footnote to History: 'The Reservoir Would Not Hold Water," Idaho Yesterdays (Spring 1980): 14.
- (Traditional routes) to Clayton Marler from Diana Yupe, Tribal Cultural Resources Coordinator, Shoshone-Bannock Tribes, June 21, 1999, copy at Cultural Resources Office, INEEL.

CHAPTER TWO

Epigraph: Kay Lambson, interview with author, February 9, 1999.

- 1. Wyle Laboratories, Interim Ordnance Cleanup Program Record Search Report, hereafter, Scientech Report (Norco, California: Scientific Services and Systems Group, 1993), Reference 2, "History of the Naval Ordnance Plant, Pocatello, Idaho," p. 1. This report reprints documents found in Naval Historical Center and other archives. See also Arrowrock Group, The National Environmental and Engineering Laboratory: A Historical Context and Assessment, (hereafter Context Report) INEEL/EXT-97-01021, Revision 2 (Idaho Falls: INEEL, 1997), 29-30; and United States, Building the Navy's Bases in World War II: History of the Bureau of Yards and Docks and the Civil Engineer Corps, 1940-1946, Vol. 1 (Washington D.C.: Government Printing Office, 1947), 1-13.
- 2. Context Report, 29-30.
- 3. Context Report, 30.
- 4. Scientech Report, Reference 2, pp. 4-5.
- 5. Context Report, 30.
- 6. *Context Report*, 31. The letters on the concrete were still visible in 1999.
- Margaret and Orville Larsen, interview with author, March 19, 1999; *Scientech Report*, p. 3-2 (road names).
- 8. Kay Lambson, Gloria Lambson, interviews with author, February 9, 1999; and *Scientech*

- *Report* Reference 1, "Register of Pertinent Information," p. 2.
- 9. Scientech Report, Reference 2, p. 22; Reference 85, p. 135; and map of Naval Proving Ground, p. 2-8. See also 1951 INEEL photo no. 02974; map U.S. Naval Proving Ground.
- 10. Stan Coloff, "The High and Dry Navy: World War II," Philtron (October 1965): 3, reprinted as "WWII: The Arco Naval Proving Ground" in INEL News (May 1989): 18; Scientech Report, p. 3-2 (road names), Reference 2, p. 5 (Scoville). Extant buildings from the Naval Proving Ground era in 1999 included the commanding officer's house (CFA-607), the Marine barracks (CFA-606), the pumphouse (CFA-642), a brick-veneer cottage (CFA-613), and a wood frame cottage (CFA-603). Three of the magazines were still in use: CFA-635 and -637 stored hazardous materials; and CFA-638 was used as a Dosimetry Calculation Laboratory.
- 11. Coloff, 3.
- 12. Gloria Lambson, February 9, 1999.
- Gloria Lambson, February 9, 1999;
 Scientech Report, Reference 1, p. 6 (buoy repair).
- Kay Lambson, Gloria Lambson, February 9, 1999
- 15. Scientech Report, p. 2-6, 2-9, 2-10, 6-1.
- 16. One designated area was about five miles northwest of the INEEL's Radioactive Waste Management Complex; the other centered on what is today's U.S. Highway 20 between East Butte and the site of EBR-II. See Scientech Report, Reference 96, p. 2-74, 6-7.
- (Quote, air-gap) to Captain M.A. Sawyer from Captain Walter E. Brown, August 11, 1944, in Scientech Report, p. 2-23 to 2-25.
- 18. Scientech Report, p. 2-22, 2-29, 2-35 to 2-38.

- Scientech Report, p. 6-7, References 76-80, 82.
- 20. Scientech Report, p. 2-31.
- 21. Scientech Report, Reference 92.
- 22. *Scientech Report*, Reference 67, p. 7; Reference 88.
- 23. Depleted uranium was a byproduct of uranium enrichment plants at Oak Ridge, Tennessee. It contained less than the natural amount of the isotope of uranium-235. The material has several uses. In breeder reactors, it can be bred into Plutonium-239 and then fissioned. It is used in armor-piercing shells, in tank armor, and for counterbalances on aircraft control surfaces. For Elsie, Marsh see *Scientech Report*, p. 2-72.

CHAPTER THREE

- Edward R. Landa and Terry B. Councell, "Leaching of Uranium from Glass and Ceramic Foodware and Decorative Items," *Health Physics* 63 (No. 3, September 1992): 343.
- Raymond W.Taylor and Samuel W.Taylor, *Uranium Fever; or No Talk Under \$1 Million* (New York: Macmillan, 1970), 79.
- 3. In some reactors, such as the fast-fissioning EBR-II, some U-238 atoms will fission.
- 4. Norman Polmar and Thomas B. Allen, *Rickover, Controversy and Genius, A Biography* (New York: Simon and Schuster, 1982), 118-120. Enrico Fermi observed uranium fission in a 1934 experiment. He thought the fission products were new elements and did not recognize them as krypton and barium. Four years later, Austrian physicist Lise Meitner realized that the uranium atom had actually split apart.
- William Benton, *The Annals of America*, Volume 15 (Chicago: The Encyclopedia Britannica, Inc., 1968), 601-602.
- Rhodes, The Making of the Atomic Bomb
 (New York: Simon and Schuster, 1986), 427;
 Stephane Groueff, Manhattan Project: The
 Untold Story of the Making of the Atomic
 Bomb (New York: Little, Brown, and Co.,
 Bantam, 1967), 55-56; Taylor, Uranium
 Fever, 81; Polmar and Allen, Rickover, 120.
 The first two of these sources and Richard
 G. Hewlett and Oscar E. Anderson, Jr., The
 New World, 1939-1946 (Philadelphia:
 Pennsylvania State University Press, 1962)
 are histories of the Manhattan Project.

- Rhodes, Atomic Bomb, 711, 740. Estimates on TNT yield equivalent differ. Rhodes' source was the Committee for the Compilaton of Materials on Damage Caused by the Atomic Bombs in Hiroshima and Nagasaki, p. 21, in Hiroshima and Nagasaki (Basic Books, 1977, 1981).
- Jack M. Holl, Argonne National Laboratory, 1946-1996 (Urbana: University of Illinois Press, 1997), 40.
- 9. See Holl, Argonne, 62.
- (Groves approval) Holl, Argonne, 40. For an account of the early deliberations of the AEC, see Richard G. Hewlett and Francis Duncan, Atomic Shield, 1947/1952
 (University Park: Pennsylvania State University Press, 1969).
- 11. Atomic Energy Act of 1954 (Public Law 79-585)
- For a history of early corporate interest in nuclear energy, see Mark Hertsgaard, Nuclear, Inc., The Men and Money Behind Nuclear Energy (New York: Pantheon Books, 1983).
- See Hewlett, Atomic Shield; Herbert York, The Advisors, Oppenheimer, Teller, and the Superbomb (San Francisco: W.H. Freeman and Company, 1976); and Herbert York, Race to Oblivion (New York: Simon and Schuster, 1970).
- 14. David Lilienthal, *Atomic Energy, A New Start* (New York: Harper and Row, 1980), 1.
- 15. John Tierney, "Take the A-Plane: The \$1 Billion Nuclear Bird that Never Flew," Science 82 (Jan/Feb 1982): 47; Henry W. Lambright, Shooting Down the Nuclear Airplane (Syracuse, NY: Inter-University Case Program No. 104, 1967), 3; Susan M. Stacy, Idaho National Engineering Laboratory: Test Area North, Hangar 629, Historic American Engineering Record ID-32-A (Idaho Falls: Lockheed Martin Idaho Corporation, 1994), 10.
- Susan M. Stacy, Idaho National Engineering Laboratory, Army Reactor Experimental Area, Historical American Engineering Record ID-33-D (Idaho Falls: Lockheed Martin Idaho Corporation, 1998), 5.
- 17. Hewlett, Atomic Shield, 196, 202-204. In 1948 the AEC research laboratories were Argonne, Chicago; Oak Ridge, Tennessee; Brookhaven, New York; and the new Knolles laboratory in Schenectady. The Los Alamos Science Laboratory in New Mexico focused on weapons research. The Radiation

- Laboratory at Berkeley, California, did not carry the title of "national laboratory." Other AEC production centers made plutonium and other weapons materials.
- Pike quoted by Kevin Richert in "Original AEC Site Spawned Eastern Idaho's `Gold Rush,'" (Idaho Falls) *Post-Register*, May 15, 1994, H-20; (Hafstad) Hewlett, *Atomic* Shield. 210.
- 19. Hewlett, Atomic Shield, 196, 206.
- 20. Hewlett, Atomic Shield, 210.
- Smith, Hinchman & Grylls, Survey on Fort Peck, Montana, and Pocatello, Idaho, Sites (Detroit: Smith, Hinchman & Grylls, March 1949), 1, 41 (fog).
- 22. Hewlett, Atomic Shield, 218.
- Robert Smylie, interview with author, July 8, 1998, Boise; "Idaho Termed Top Favorite for Plant Site," (Idaho Falls) *Post-Register*, March 16, 1949. 1.
- 24. "Arco Area to Get West Atomic Plant," Post-Register, March 22, 1949, 1. See also letter to Mike Mansfield from David Lilienthal, March 29, 1949, IHS AR 2/22, Papers of Governor C.A. Robins, Box 1, Series 1, File: Arco Atomic Energy Plant/part 1; (hearings) Hewlett, Atomic Shield, 211.
- 25. "Station Manager Named," *Post-Register*, April 4, 1949, 1.

CHAPTER FOUR

- "Who cares for letterheads?" Arco
 Advertiser, April 29, 1949, 1; "Arco Chief
 Wonders if Neighbor is Lacking in Brotherly
 Love," Idaho Daily Statesman, April 29,
 1949; "Idaho Falls Pulls Back on Publicity
 as Atomic Capital," Post-Register, April 26,
 1949.
- "Station Manager Named," Post-Register, April 4, 1949, 1.
- Because of the Navy's resistance to giving up the proving ground, the AEC discussed other locations for the MTR, EBR, and Navy reactors as late as mid-July; Hewlett, Atomic Shield, 218.
- 4. Jacqui Johnston Batie, interviews with author on October 14, 1998, and March 14, 1999; Bill Ginkel, interviews with author on October 13, 1998, and February 3, 1999; and undated news clipping supplied by Jacqui Batie, "Leonard E. Johnston, AEC Chief Here Takes New Job, Reports Say."

Dolts

- See Groueff, *Manhattan Project*, 381, for gaseous diffusion plant.
- "Details Revealed for Plan to Build West Atomic Field Station," Post-Register, March 8, 1949, 1; "Arco Site Named for Atomic Plant," Arco Advertiser, March 25, 1949, 1; and "Leonard E. Johnston, AEC Chief Here Takes New Job, Reports Say," (see note no. 4 above).
- "A Village Wakes Up," Life (May 9, 1949): 98-101; "Arco Site Named for Atomic Plant," Arco Advertiser, March 25, 1949, 1. See also Vardis Fisher, ed. The Idaho Encyclopedia (Caldwell, Idaho: Caxton Printers, 1938) entry for Arco.
- "Village Alert to Problems" and "New Business Firms Move In," Arco Advertiser, 1; to Chet Moulton from Governor Robins, April 14, 1949, IHS AR2/22, Box 1, Series 1, File: Arco Atomic Energy Plant, part 1; and Federal Housing Administration, "A Preliminary Plan and Program for Arco, Idaho," 1949, in AR2/22, File: Arco Atomic Energy Plant, part 3.
- 8. (Quote) to Robins from Cy Davis, Idaho Falls Chamber of Commerce, March 28, 1949, IHS AR2/22, Box 1, Series 1, File: Arco Atomic Energy Plant, part 1.
- (Quote) Robins from Cy Davis, March 28, 1949, cited in note above; to Robins from Joe Call, Idaho Falls Chamber of Commerce, March 28, 1949, IHS AR2/22, Box 1, Series 1, File: Arco Atomic Energy Plant, part 1. The towns were Island Park, Warm River, Ashton, St. Anthony, Rexburg, and Arco.
- "Mayors Form Committee to Cooperate with Atomic Project," *Arco Advertiser*, April 8, 1949, 1.
- 11. Telegram to Robins from John Sanborn, April 12, 1949, IHS AR2/22, Box 1, Series 1, File: Arco Atomic Energy Plant, part 1.
- 12. "Who Cares for Letterheads?" *Arco Advertiser*, April 29, 1949, 1.
- 13. Robb Brady, interview with author, August 27, 1998.
- 14. Robb Brady, August 27, 1998.
- 15. "Blackfoot eyes Possibility of Atomic Future," *Post-Register*, April 3, 1949, 5. See also Ben Plastino, *Coming of Age: Idaho Falls and the Idaho National Engineering Laboratory* (Idaho Falls: Margaret A. Plastino, 1998), 7-8.

- 16. Marvin Walker, interview with author, March 15, 1999. Walker, Johnston's driver and document courier, heard Johnston remark several times that the NOPwould have been an ideal set-up for the AEC headquarters, except for the lack of commitment from Pocatello. See also Plastino, Coming of Age, 17.
- 17. AEC news release, May 18, 1949, IHS AR2/22 Box 1, Series 1, File: Atomic Energy Plant, part 1.
- David Lilienthal, The Atomic Energy Years, 1945-1950, The Journals of David E. Lilienthal, Volume II (New York: Harper and Row, 1964), 513-514.
- 19. "Purpose of Arco Site Explained," *Arco Advertiser*, August 5, 1949, 1.
- Jacqui Batie, Marvin Walker, and John Ray, interviews with author, March 14 and 15, 1999.
- 21. "Problems Mount as City Squares Off to Big Job," *Post-Register*, May 22, 1949, 4.
- 22. R.L. Polk and Co., *Polk's Idaho Falls City Directory* (Omaha: R.L. Polk, 1952), 13.

CHAPTER FIVE

Epigraph: Bill Johnston to C.A. Robins, July 13, 1949. See note no. 1 below.

- "AEC Offices in New Location," Arco Advertiser, July 22, 1949, 1; letter to C.A. Robins from Bill (L.E.) Johnston, July 13, 1949, IHS AR 2/22, Box 1, Series 1, File: Arco Atomic Energy Plant, part 2; (bubble rumor) Hewlett, Atomic Shield, 219.
- 2. Hewlett, Atomic Shield, 218-219.
- 3. Holl, Argonne, 87.
- "Gradual Expansion of Arco Plant Planned by AEC," Arco Advertiser, September 30, 1949. 1.
- Marvin Walker, interview with author, March 15, 1999; Hewlett, *Atomic Shield*, 495; William Ginkel, interview with author, February 3, 1999.
- Jack M. Holl, "The National Reactor Testing Station: The Atomic Energy Commission in Idaho, 1949-1962," *Pacific Northwest Quarterly* (Volume 85, No. 1, January 1994): 18.
- 7. John Horan, interview with author, July 29, 1997.
- 8. "Exploratory Surveys Start at Atomic Site," *Arco Advertiser*, July 1, 1949, 1.

- John Horan, interview with author, July 29, 1997.
- 10. John Horan, July 29, 1997.
- 11. Smith, Hinchman & Grylls, Survey, 54, 70, 80. Harold T. Stearns and J. Stewart Williams, consulting geologists, were subcontracted by Smith, Hinchman & Grylls to summarize their understanding of the aquifer. See "Geological and Hydrologic Features of the Pocatello Site, Idaho," in the Survey.
- "Gradual Expansion of Arco Plant Planned by AEC," Arco Advertiser, September 30, 1949, 1; Hewlett, Atomic Shield, 211; "Testing Proves Ample Supply," Arco Advertiser, August 19, 1949, 1.
- 13. Marvin Walker, March 15, 1999.
- Hewlett, Atomic Shield, 218-220. For a history of the hydrogen bomb, see Richard Rhodes, Dark Sun, The Making of the Hydrogen Bomb (New York: Simon and Schuster, 1995).
- "Gradual Expansion of Arco Plant Planned by AEC," Arco Advertiser, September 30, 1949, 1.
- 16. Rick Bolton, "USGS technician Rodger Jensen reflects on 30 years in the sagebrush" INEL News (May 19, 1992); "A Village Wakes Up," Life (May 9, 1949): 101; (Johnston quote) "Purpose of Arco Site Explained," Arco Advertiser, August 5, 1949, 1.
- 17. "AEC Given Custody of Naval Proving Ground," *Arco Advertiser*, December 9, 1949. 1.
- Hewlett, Atomic Shield, 495; "Fresh Water Test Well Contract Bid" and "Batch Plant Equipment Moved to Atomic Site," Arco Advertiser, October 7, 1949, 1.
- 19. Deslonde deBoisblanc, interview with author, January 16, 1999.
- 20. Marvin Walker, March 15, 1999.
- 21. To Robins from Johnston, August 24, 1949, and September 9, 1949; to Johnston from Robins, September 2, 1949; all in IHS AR2/22, Box 1, Series 1, Arco AE Plant—1949, part 1.
- 22. To James Reid, Idaho Bureau of Highways, from J. Bion Philipson, November 22, 1949, IHS AR2/22, Box 1, Series 1, File: Arco Atomic Energy Plant 1949, part 1.
- 23. (Robins quotation) to Johnston from Robins, November 30, 1949, IHS AR2/22, Box 1,

- Series 1, File: Arco Atomic Energy Plant, 1949, part 1; also see letter to J. Bion Philipson from James Reid, November 30, 1949, IHS AR2/22, Box 1, Series 1, File: Arco Atomic Energy Plant 1949, part 1.
- 24. "Dworshak and Welker in Blackfoot Today, Discussion on Highway 20 Construction; Irate as Johnson Bypasses the Meeting," Blackfoot Daily Bulletin, September 12, 1952.
- 25. "Groups Ready Dedication of Idaho Falls-Arco Highway," *Post-Register*, September 21, 1951; "Deal Dedicates New Arco, Idaho Falls Highway," *Post-Register*, October 8, 1951.
- 26. (Quotation) Lou Haller, "Johnston Explains AEC Position on the Road, Rich Sets 1954-'55 As Completion Date; Senator Dworshak Has Asked Federal Aid," *Blackfoot Daily Bulletin*, September 23, 1952; "Dworshak and Welker in Blackfoot Today; Discussion on Highway 20 Construction; Irate as Johnston By-passes the Meeting," *Blackfoot Daily Bulletin*, September 12, 1952; Plastino, page 13.
- 27. "State Engineer Says Highway 26 Serving AEC from Blackfoot Should Be Given First Priority," *Blackfoot Daily Bulletin*, November 21, 1952; "Sen. Dworshak Reports AEC and Bureau of Public Roads Have Assured Construction Money Aid," *Post-Register*, December 22, 1952.
- 28. "Sen. Dworshak Reports AEC and Bureau of Public Roads Have Assured Construction Money Aid," *Post-Register*, December 22, 1952; "Crews Start Plotting AEC-Rexburg Road," *Salt Lake Tribune*, January 27, 1952.
- 29. (Bus fare) William Ginkel, interview with author, February 3, 1999.
- 30. To Alton B. Jones, State Superintendent of Schools, from George Green, Superintendent of Schools for Bannock County, April 26, 1949, IHS AR2/22, Box 1, Series 1, File: Arco Atomic Energy Plant, part 1. See also Holl, *The National Reactor Testing Station*; Robb Brady, interview with author, August 27, 1998.
- "L.E. Johnston Explains Purpose and Development of Reactor Plant," *Arco Advertiser*, December 9, 1949, 1.

CHAPTER SIX

Epigraph: The occasion for the song was an AEC announcement that Oak Ridge would not be involved further in reactor development, but that such work would be centralized at Argonne. See Hewlett, *Atomic Shield*, 126.

- Kirby Whitham, interview with author, October 14, 1998. The test was conducted in the EBR-I reactor. An organic compound is one which contains carbon.
- 2. William Ginkel, interview with author, February 3, 1999.
- 3. Holl, Argonne, 68-69.
- In reactor technology, neutrons with energies above 100,000 electron volts are called fast neutrons. EBR-I neutrons moved with two million electron volts of energy.
- For a general overview of fast reactors see Alan M. Jacobs, et al, *Basic Principles of Nuclear Science and Reactors* (D. Van Nostrand Company: Princeton, 1960), 171-175.
- "New Pump to Aid AEC At Idaho Falls Plants," Salt Lake Tribune, April 14, 1952. Engineers also installed more conventional pumps in each of the main cooling systems.
- 7. Hewlett, Atomic Shield, 496.
- 8. Kirby Whitham, October 14, 1998.
- "Dr. Walter Zinn Recalls Early Days," *Idaho National Engineering Laboratory EBR-I Review*, 1. This undated reprint with no other publication data is a collection of stories about EBR-I printed some time after December 13, 1971, possibly by the INEL employee newspaper. Copy found in Bechtel Cultural Resources departmental files at INEEL. Hereafter, *EBR-I Review*.
- 10. John H. Buck and Carl F. Leyse, Materials Testing Reactor Project Handbook, Report No. TID-7001 (Lemont, Illinois: Argonne National Laboratory and Oak Ridge National Laboratory, 1951), 356. For a brief description of nuclear physics research at the MTR, see NRTS, 1965 Thumbnail Sketch, 16-17. Beam experiments did extend outside the MTR building, but not to distances of a quarter of a mile.
- 11. Buck and Lehse, MTR Handbook, 29.
- 12. For general history of MTR development, see Buck and Lehse, *MTR Handbook*, 34-43.
- 13. Donald J. Hughes. *The Neutron Story* (Garden City, N.Y.: Doubleday Anchor

- Books, 1959), 41; Buck and Leyse, MTR Handbook, 29.
- 14. Buck and Lehse, *MTR Handbook*, 37; Hewlett, *Atomic Shield*, 126, 419.
- 15. Hewlett, Atomic Shield, 186-188.
- (Groundbreaking) Buck and Lehse, MTR Handbook, 43; (siting) John Horan, July 29, 1997.
- 17. Hewlett, Atomic Shield, 496.
- (Quotation in previous paragraph and extended quote) John W. Simpson, *Nuclear Power from Underseas to Outer Space* (La Grange Park, Illinois: American Nuclear Society, 1995), 22 and 26-27, respectively.
- Simpson, Nuclear Power, 27; (MTR contribution to STR) 1965 Thumbnail Sketch, 13.
- Accounts of Rickover and the Nuclear Navy include Richard Hewlett and Francis Duncan, Nuclear Navy 1946-1962 (Chicago: University of Chicago Press, 1974); Polmar and Allen, op cit.
- 21. (Rickover quote) Ronald Schiller, "Submarines in the Desert," *Colliers Magazine* (February 5, 1954): 90. Polmar and Allen, *Rickover*, 149. The original name of the Idaho reactor was Mark I; the *Nautilus* reactor, Mark II. Rickover referred to the development shortcut as "Mark I equals Mark II."
- 22. Simpson, Nuclear Power, 43.
- 23. (McGaraghan's Sea) Schiller, Colliers, 88; (backscatter) Dr. John Taylor, communication with author, August 6, 1999. Taylor was responsible for the backscatter research.
- The Bluejackets Manual, 19th edition (Annapolis, Maryland: Naval Institute Press, 1973), 153.
- 25. Simpson, Nuclear Power, 27, 224-337.
- 26. Simpson, Nuclear Power, 50.

CHAPTER SEVEN

Epigraph: Marion Thomas, "Recollections and Reflections," *Idaho American Nuclear Society Newsletter* (June 1989), 5.

- Deslonde R. deBoisblanc, *Idaho ANS Newsletter*, 5-6. This issue contained several tributes to Dr. Doan by former associates.
- 2. Fred McMillan, interview with author, February 2, 1999.

DOLES

- 3. deBoisblanc, ANS Newsletter, June 1989.
- (Architects) Fred McMillan, February 2, 1999; (welcome) see for example "Two Join Phillips Petroleum Staff," *Post-Register*, August 1, 1951, 3.
- "Atom Pioneer Directs Phillips Project Here," Post-Register, August 19, 1951. See also Virginia Doan Fanger, ANS Newsletter, June 1989.
- 6. Stephane Groueff, *Manhattan Project*, 80.
- deBoisblanc, *Idaho ANS Newsletter*, June 1989; Richard Austin Smith, "Phillips Petroleum—Youngest of the Giants," *Fortune* (August 1954): 72; and Phillips Petroleum Company, *Phillips, The First 66 Years* (Bartlesville, Oklahoma: Phillips Petroleum Company, 1983), 86, 125-140.
- 8. Warren Nyer, *Idaho ANS Newsletter*, June 1989, 7-9.
- 9. Ibid.
- (Pappy) John Byrom, interview with author, August 25, 1999; deBoisblanc, Miles Leverett, Warren Nyer, ANS Newsletter, June 1989, 6-9.
- 11. Phillips, The First 66 Years, 86, 125-140.
- 12. The study is described by W. Singlevich, et al, Natural Radioactive Materials in the Arco Reactor Test Site, Report No. HW-21221 (cover title: Ecological and Radiological Studies of the Arco Reactor Test Site (Richland: General Electric Nucleonics Division, Hanford Works, 1951), 1-49. See also "AEC Sets Radioactivity Study at Idaho Site; College Gets Contract," Post-Register, June 19, 1950, 3.
- 13. John Horan, in an interview with author, indicated that Health and Safety Division personnel in particular made use of the report: Singlevich, et al, Natural Radioactive Materials in the Arco Reactor Test Site, Report No. HW-21221. See note 12 for full citation.
- 14. (Nevada test site) Hewlett, Atomic Shield, 535; (malfunctioning, 10 mr/hr on truck) Phillips Petroleum Company, Survey of Fallout of Radioactive Material in South and South-East Idaho Following the Las Vegas, Nevada Tests of October and November, 1951, Report IDO 12000 HPO (Idaho Falls: Site Survey Section of the Health Physics Division, NRTS, 1952), 11, 8; (truck, rainstorm explanation) John Horan, "Draft

- of Interview" with John K. Harrop and Joseph J. Shonka in Idaho Falls, Summer 1994, 2.
- See for example, "Press Release, May 18, 1961: Annual Summary of Environmental Monitoring Data for 1960," in records of Senator Henry Dworshak, IHS, Ms. 84, Box 122 B, File: AEC-Id-Press Releases.
- 16. "AEC to Employ Top Safeguards Here," *Post-Register*, May 11, 1950, 15.
- 17. Henry Peterson, interview with author, February 22, 1999; (NCRPstandards) Elizabeth S. Rolph, *Nuclear Power and the Public Safety* (Lexington, Mass.: D.C. Heath and Co., 1979), 108; John Horan, interview with Baldwin and Bacchus, 1994. An exposure of 50 milirems per day was an allowable exposure, but alarms sounded at 80 percent of that value.
- "AEC Sets Radioactivity Study at Idaho Site; College Gets Contract," *Post-Register*, June 19, 1950, 3; "Experts Test Weather Data," *Post-Register*, May 14, 1950, 14.
- 19. Idaho Department of Labor, First Biennial Report of the Department of Labor, August 15, 1949—December 31, 1950, 1.
- To Governor Robins from W.L. Robison, March 22, 1950, IHS AR2/22, Box 1, File: Arco Atomic Energy Plant-1950, part II. See also "AEC to Organize Medical Program," Post-Register, June 21, 1950, 9.
- (Accident rates) First Biennial Report, 25;
 (ceremony) Idaho Department of Labor,
 Third Biennial Report of the Department of Labor, October 31, 1952 to October 1954,
 29.
- 22. "Committee to Advise AEC on Health in Area," Arco Advertiser, March 19, 1954, 3; "Idaho Board Approves AEC Health Safeguards, Arco Advertiser, March 26, 1954, 1; "State Committee Ends Meeting at AEC Plant," Arco Advertiser, July 22, 1955, 7.
- 23. (Hospitals) Ninth Semiannual Report of the AEC, January 1951, 31; (Idaho hospitals) Dr. Lawrence Knight, interview with author, April 20, 1999; "Mountain States Tumor Institute," supplement in Idaho Statesman, May 11, 1997, 2. St. Lukes Hospital in Boise organized an isotope laboratory in 1957 and began cobalt cancer therapy in 1960. (Shoes) Department of Health, Radiation Health Section, "Program for the Regulation and Control of the Use of Radiation in the State of Idaho, 1968," 5,

- IHS, Samuelson Papers, Box 50, File: Governor Samuelson Nuclear—Misc.
- 24. "Minutes of the Idaho State Board of Health, April 21-22, 1958; August 4-5, 1958," Minute Book at Idaho State Department of Health and Welfare, Director's Office, Boise, Idaho, p. 58 and 63, respectively. Among early industrial users of radioactive materials was the agricultural industry, which used tracers to help identify the uptake of pesticides or fertilizers in plants.
- 25. "Program for Regulation and Control... 1968," 5.
- 26. See, for example, letter to Smylie from Lyall Johnson, Acting Director of the Division of Materials Licensing, AEC-DC, May 13, 1964, in IHS AR2/23, Box 17, File: Atomic Energy.
- 27. To Governor Smylie from Terrell Carver, August 9, 1963, IHS, Smylie Papers, Box 20, File: Health Department, 1963. See also Minutes of the Board of Health, July 30-31, 1964, p. 241. The IDO/Phillips representatives were C. Wayne Bills, J. Weaver McCaslin, and Tabb O'Brien; "Minutes of the Governor's Committee on the Use of Atomic Energy and Radiation Hazards, February 19, 1964," held at the office of the Department of Labor, Boise. In IHS, Smylie Papers, Box 21, File: Labor Department 1962-1964.
- 28. To Clinton P. Anderson from T.O. Carver, February 18, 1959; IHS, Mss 84, Box 104, File: Legislative AEC Misc. 1959.
- 29. "Statement by the Honorable Robert E. Smylie, Governor of the State of Idaho, to the Natural Resources Subcommittee of the Committee on Government Operations," presented at a hearing held November 22, 1963, Seattle, Washington, IHS, AR2/23, Box 25, File: Len Jordan 1962-64. For a review of Idaho/AEC relations regarding housing, grazing, law enforcement, and other matters, see Jack M. Holl, "The NRTS: The AEC in Idaho," 85 Pacific Northwest Quarterly (January 1994): 15-24.

CHAPTER EIGHT

Epigraph: Idaho Department of Labor, 2nd Biennial Report, 21. Thanks to Warren Nyer, from whom I first heard the term "reactor zoo."

 Hewlett, *Atomic Shield*, 497. See also, Holl, *Argonne National Laboratory*, 1946-1996,
 Holl reports the shortage as 12.5 kilograms of uranium-235, this being the

- amount over Zinn's original estimate of 40 kilograms exclusive of the fuel in the extra fuel rods.
- Kirby Whitham, communication to author, July 9, 1999. According to Whitham, the remanufactured rods were .010 inches larger in diameter.
- 3. Hewlett, Atomic Shield, 497-98. The significance of the names on the wall is uncertain. According to Richard Lindsay of Argonne-West, not all of the crew members were present during the chalk ceremony; a few added their names later but others did not. The sensibilities of the era were such that the women who ran the EBR office were not included. In 1995 the names of the women were placed on a plaque and installed in the EBR-I building, but the names of missing men have not been so identified. See IDO News Release, March 20, 1995, and "Better late than never for EBR-I women," Post-Register, March 24, 1995.
- "Reactor Staff Weary, Happy" and "Materials Testing Reactor Achieves Operation Stage Here," *Post-Register*, April 4, 1952, 1; and Fred McMillan, interview with author, February 2, 1999.
- 5. (Squirrels) Jack Clark, interview with author, July 16, 1998; (Doan) John Byrom, August 25, 1998. "Scram" was a slang word meaning, "let's get out of here." Its use as a term for emergency reactor shutdowns originated with CP-1, the first reactor. For safety backup, a man was stationed near a rope restraining a control rod. Should other controls fail, he was to cut the rope, letting the rod fall into place. Someone devised an acronym "Safety Control Rod Ax Man." Another acronym, according to Polmar in *Rickover*, p. 151, was "Super-Critical Reactor Ax Man."
- John Byrom, August 25, 1998; Phillips Petroleum Company, The Materials Testing Reactor, A Light Water Moderated Reactor (Geneva, Switzerland: International Conference on Peaceful Uses of Atomic Energy, 1955) "Research Reactors," p. 352.
 See also Buck and Leyse, MTR Handbook, 352.
- 7. Buck and Leyse, MTR Handbook, 29-30.
- 8. John Byrom, interviews with author, August 25, 1998, and March 26, 1999. See also E. Fast, J.P. Byrom, and J.W. McCaslin, *A Survey of the Materials Testing Reactor Shield*, Report No. (IDO-16073) (Idaho

- Falls: Phillips Petroleum Co., n.d.) and Blaw Knox Construction Company, *Barytes Aggregate Concrete Applied to Reactor Shielding*, Report No. IDO-24003 (Idaho Falls: Blaw Knox Chemical Plant Division, 1952).
- 9. John Byrom, personal communication to author, October 29, 1998.
- 10. Fred McMillan, interview with author, February 2, 1999.
- 11. John Taylor, personal communication to author, August 6, 1999.
- 12. Fred McMillan, February 2, 1999.
- 13. Simpson, Nuclear Power, 47.
- 14. Simpson, Nuclear Power, 53.
- 15. Schiller, Colliers, 91.
- 16. Polmar and Allen, Rickover, 151-152.
- 17. Polmar and Allen, Rickover, 152.
- 18. Simpson, Nuclear Power, 55-56.
- 19. Polmar and Allen, Rickover, 153.
- 20. The original telegram is located in a Naval Reactors Facility historical scrapbook, 1958.
- 21. Polmar and Allen, Rickover, 615.
- 22. By the time the Navy laid the keel for USS Enterprise on February 4, 1958, it had launched or laid the keel for the submarines Nautilus, Skate, Seawolf, Swordfish, Sargo, Seadragon, and Triton; and the cruiser USS Long Beach.

CHAPTER NINE

- "Industrial Radioactive Waste Disposal,"
 Hearings before the Special Subcommittee
 on Radiation of the JCAE 86th Cong, 1st
 session, Volume 1, January 28, 29, and 30;
 February 2 and 3, 1959 (Wash, D.C.: GPO,
 1959). Hereafter, Radioactive Waste
 Hearings.
- (Quotation) Rafe Gibbs, "The World's Hottest Garbage," *Popular Mechanics* (April 1955): 124-125; (Argonne) Holl, *Argonne*, 57-58.
- 3. Holl, Argonne, 73.
- B.C. Anderson, et al., A History of the Radioactive Waste Management Complex at the Idaho National Engineering Laboratory (Idaho Falls: IDO Nuclear Fuel Cycle Division, 1979), 1. Hereafter, History of the RWMC.
- 5. Anderson, *History of the RWMC*, 2; (precipitation) 1971 *Thumbnail Sketch*, 3.

- 6. Anderson, History of the RWMC, p. 4, 16.
- 7. Henry Peterson, interview with author, February 3, 1999.
- 8. Henry Peterson, February 3, 1999; (diapers) Ronald Schiller, *Colliers*, 90; (sanitary pads) Beverly Cook, interview with author, September 7, 1999.
- Edgar Juell, A Short History of the Expended Core Facility, 1953 to June 1990 (Idaho Falls: Naval Reactors Facility Expended Core Facility, no date), 1.
- (General procedures, liquids) Anderson, History of the RWMC, 16; (irradiated metal parts) Doan remarks in Radioactive Waste Hearing, 601.
- 11. (Native grasses) Byrom, personal communication; (practices) Anderson, *History of the RWMC*, 16. In current practice, cardboard boxes are no longer used. Waste is packaged in wooden boxes or metal drums, each lined with plastic.
- 12. For Rocky Flats history, see www.indra.com/rfcab/ FAQ.html#4.
- Anderson, History of the RWMC, 19. The Rocky Flats area was later expanded to eleven square miles. Source: www.cdphe.state.co.us, April 26, 1999.
- 14. (Nevada/Idaho) Anderson, *History of the RWMC*, 19.
- 15. John Horan, interview with author, January 15, 1999.
- 16. John Horan, January 15, 1999. See also Anderson, *History of the RWMC*, 21.
- 17. Radioactive Waste Hearing, 582-583.
- 18. Anderson, History of the RWMC, 21-22. Also, see photo 58-1450. See "Alpha Decay" in John F. Hogerton, The Atomic Energy Deskbook (New York: Reinhold Publishing Corp., 1963), 21. Plutonium is highly ionizing and presents a hazard to human health when it is inhaled or ingested.
- Clyde Hammond, interview with author,
 October 15, 1998. According to D.H. Card,
 Waste Management Program History of
 Buried Transuranic Waste at INEL, (Idaho
 Falls: EG&G Report No. WMP77-3, 1977),
 9, Gulf Atomic sent 190 drums and 22 boxes
 of waste to the NRTS in 1962. Some drums
 were centered in boxes, and the boxes filled
 with concrete for shielding. These weighed
 between 12,000 to 14,000 pounds each.

Nole

- Cass Peterson, "Rocky Flats: Risks Amid a Metropolis," Washington Post, December 12, 1988, 1.
- 21. Anderson, History of the RWMC, 22.
- 22. Clyde Hammond, October 15, 1998.
- 23. (Backfill) John Commander, personal communication to author, June 29, 1999; (strike) Anderson, *History of the RWMC*, 31; (efficiency) George Wehmann, interview with author, September 29, 1999.
- 24. Anderson, History of the RWMC, 28. According to EG&G, Waste Management Program, History of Buried Transuranic Waste at INEL, (Idaho Falls: EG&G Report No. WMP77-3, 1977), p. 8, waste arriving at this time included contaminated materials from other governmental experimental sites around the nation.
- (Upper limits, flooding, guidelines, Interim Burial Ground) Anderson, *History of the* RWMC, 19, 33, 30, 27.
- 26. Anderson, *History of the RWMC*, 30; (quotation) *Radioactive Waste Hearing*, 584.
- 27. Anderson, History of the RWMC, 35.
- 28. Formal definitions of "high-level," "lowlevel," TRU, spent fuel, and other categories of radioactive waste have changed over time and have become more complex; recent definitions are found in the Energy Policy Act of 1992, Title 10 of the Code of Federal Regulations (CFR), Part 60 and in DOE Order 5820.2A. At the INEEL, high-level waste refers to the solid and liquid wastes resulting from chemical separations done at the Idaho Chemical Processing Plant. TRU waste contains alpha-emitting transuranic elements (like plutonium-contaminated waste from Rocky Flats) with half-lives of 20 years or more and concentrated at 100 nanocuries per gram of waste or more. Lowlevel waste, typically disposed or stored at the Burial Ground, in general contains lower concentrations of radionuclides per gram of waste than high-level and TRU waste. See DOE, Linking Legacies Report No. DOE/EM-0319 (Washington, D.C.: DOE Office of Environmental Management, 1997), pp. 31-70.
- 29. (Handbook) *Radioactive Waste Hearing*, 581. For the general AEC philosophy on concentrating and reducing volumes of waste, see remarks by Dr. Abel Wolman in the hearing on p. 8-10.
- 30. Radioactive Waste Hearing, 581, 589.

- 31. Radioactive Waste Hearing, 581.
- 32. Whether to measure the concentration of a contaminant at the point of discharge or at the point of its use downstream later became the subject of regulatory definition.
- 33. Bruce Schmalz, interview with author, February 1, 1999. See also *Radioactive Waste Hearing*, 589.
- John Horan, "Draft of Interview, Mr. John Horan" with John K. Harrop and Joseph J. Shonka, Summer 1994, p. 4.
- 35. Radioactive Waste Hearing, 602.
- John Byrom, personal communication to author.
- 37. Arrowrock Group, Context Plan, 146.
- 38. (Pellets) Leroy Lewis personal communication to author.
- 39. See INEELphotos 56-3201, -3203, -3205, -3206, -3207, and -3208.
- 40. INEL, Idaho National Engineering Laboratory Historical Dose Evaluation, Volume 1, Report No. DOE/ID-12119 (Idaho Falls: IDO, 1991), pp. iii, v-vii, 6-8. Later DOE expenditures on the removal of waste from below ground were justified on the basis that these pathways should not ever find a human population at any time in the long-range future.

CHAPTER TEN

- 1. Polmar and Allen, Rickover, 167.
- Bettis Plant, Expended Core Facility, Maintenance and Operations Guide (Pittsburgh: Westinghouse, 1958), 3. Hereafter, ECF Guide.
- ECF Guide, v. The ECF was designed by the AEC's Bettis Atomic Power Laboratory, managed by Westinghouse. Arthur G. McKee & Co. was architect/engineer; and Paul Hardeman, Inc., general contractor.
- ECF Guide, 14. Later equipment allowed for the unloading of a cask without having to plunge the cask into the water.
- Edgar L. Juell, A Short History of the Expended Core Facility, (Idaho Falls: Naval Reactors Facility, 1990), 9-11 (excerpt p. 10).
- 6. Juell, A Short History of ECF, 12.
- 7. (Three cores) *ECF Guide*, 15; (MTR irradiated fuel) Juell, *A Short History of ECF*, 9.
- 8. Juell, A Short History of ECF, 13.

- (Early training) "Crews Train in Idaho for A-Submarine," *Idaho State Journal*, February 22, 1953. Thanks to Kenhi Drewes and Hal Paige, interviews with author, February 4, 1999, for discussions of training program.
- 10. Polmar and Allen, Rickover, 297-301.
- (Rickover quote) Polmar and Allen, *Rickover*, 302; (cross-training) Kenhi Drewes and Hal Paige, February 4, 1999.
- 12. Ronald Schiller, "Submarines in the Desert," *Colliers* (February 5, 1954): 91.
- Kenhi Drewes, Hal Paige, February 4, 1999; (cooling pond) Mary Freund, interview with author, February 2, 1999.
- 14. On aircraft carriers, cargo such as tanks of lubricating oil could be located to offer shielding opportunities around shipboard reactors. *USS Enterprise* was launched on September 24, 1960.
- 15. Clay Condit, interview with author, August 25, 1999.
- 16. To Governor from Henry Dworshak, April 8, 1961, IHS Mss 84, Box 122 B, File: AEC-Idaho Plant; to Robb Brady from Henry Dworshak, April 8, 1961, IHS, Mss 84, Box 122 B, File: AEC—Idaho Plant.
- 17. (Criticality date) Naval Reactors Facility pamphlet, no date, no page numbers. For more on the U.S. Navy's quest for silent submarines and the compromises made among silence, depth, and speed, see Tyler, Running Critical, The Silent War, Rickover, and General Dynamics (New York: Harper and Row, 1986). See Tom Clancy, The Hunt for Red October (Annapolis, Md.: Naval Institute Press, 1984) for a fictionalized account of the competition between the two superpowers; and "Submarines, Secrets, and Spies," a NOVA television program produced by the Documentary Guild for WGBH/Boston with Sveriges Television, copyright 1999.
- 18. Elizabeth S. Rolph, *Nuclear Power and the Public Safety*, 24-26.
- (Sidebar) U.S. DOE and U.S. Department of Defense, *The United States Naval Propulsion Program* (Washington, D.C.: GPO), 19.
- 20. Rolph, Nuclear Power and the Public Safety, 26.
- 21. Clay Condit, August 25, 1999.

 (Steno) Mary Freund; (preparations) Henry Peterson, February 3, 1999; (legend) Hal Paige, February 4, 1999.

CHAPTER ELEVEN

- Don Reid, in undated transcript of taped interview named "Don Reid's Early CPP History, Cassette Tape # 1," p. 9-10. A second tape was named "Cassette Tape # 2." No copies of the tape were found, nor the name of the interviewer. Transcript supplied to author by Jeff Bryant of INTEC (Chem Plant)
- 2. Don Reid, Cassette Tape # 1, 14, 16.
- (AEC plans) Don Reid, Cassette Tape # 1, 14-17. In a bomb detonation, tritium undergoes a fusion reaction with deuterium, producing a generous supply of neutrons which in turn promote the fissioning of plutonium atoms.
- TWX from C. Vanden Bulck, Oak Ridge National Laboratory, to R. Cook, USAEC, Washington, D.C., August 31, 1953, HREX Doc. No. 0709671.
- Richard Rhodes, The Making of the Atomic Bomb (New York: Simon and Schuster, 1986), 574. See also "Human Studies Project Team Fact Sheet, Bayo Canyon/The RaLa Program," January 10, 1994, HREX Doc. No. 0701316, p. 1.
- To R. Cook, Washington, D.C., from C. Vanden Bulck, acting manager ORNL, August 31, 1953, HREX Doc. no. 0709671. See also John Horan, Summer 1994 interview by John K. Harrop and Joseph J. Shonka, pp. 1-2. Iodine is taken up by the human thyroid gland.
- General Manager's Diary entry for April 18, 1950, HREX Doc. No. 0723885; to Carroll Wilson, General Manager, AEC, from George Brown, Director of Engineering, AEC, April 25, 1950, HREX Doc. No. 0720687; to George Brown from Ray Knapp, May 3, 1950, HREX Doc. No. 0720685.
- The hexone process was called REDOX; the tributyl phosphate process, PUREX. In the history of the Chem Plant, there were three accidental criticalities.
- 9. Succeeding operators were Phillips Petroleum Company, 1953-1966; Idaho Nuclear Corporation, 1966-1971; Allied Chemical, 1971-1979; Exxon Nuclear Corporation, 1979-1984; Westinghouse

- Idaho Nuclear Corporation, 1984-1994; Lockheed Martin Idaho Corporation, 1994-1999. The Bechtel Corporation assumed responsibility in 1999.
- 10. Don Reid, Cassette Tape # 2, 6-9.
- 11. Mark L. Sutton, "ICPP Historical Data," September 8, 1988, 1. Copy supplied to author by Jeff Bryant. This also is source for sidebar, p. 98, "Memories of an Operator Helper at the Chem Plant."
- R.B. Lemon and D.G. Reid, "Experience With a Direct Maintenance Radiochemical Processing Plant," Proceedings of the International Conference on the Peaceful Uses of Atomic Energy Volume 9 (Geneva: United Nations, 1956), 536.
- Don Reid, Cassette Tape No. 2, 15. See also "Uranium Production—ICPPProcessing Runs," Revised 10-1-75, MPH, an ICPP history of production runs; and "AEC Chemical Processing Plant Begins Operation at Arco Site," *Idaho State Journal*, March 15, 1953.
- 14. Reid, Cassette Tape # 2, 11.
- 15. Reid, Cassette Tape # 2, 12-14.
- 16. For a more detailed description of the ICPP's modified PUREX process, see Brewer F. Boardman, *The ICPP (A Factsheet)* (Idaho Falls: Idaho Operations Office, 1957). For a general description of the plant and its operations, see R.B. Lemon and D.G. Reid, "Experience With Direct Maintenance Plant," 532-545.
- 17. L.A. Decker, "Significant Accomplishments in the History of the ICPP," paper presented at the 1972 National Meeting of the American Chemical Society, San Francisco, California, September 1, 1976, p. 5. In 1969 Idaho Nuclear Corporation designed a denitration facility to convert the liquid into UO₃, a granular solid, for which, see N.J. Rigstad, "ICPPEnvironmental Impact Statement," 1973, a draft manuscript supplied to author by Jeff Bryant, p. 17. The change was stimulated by more stringent packaging and shipping requirements.
- 18. Dieter Knecht, et al, "Historical Fuel Reprocessing and HLW Management in Idaho," *Radwaste Magazine* (May 1997): 40. See also Richard K. Lester and David J. Rose, "The Nuclear Wastes at West Valley, New York, *Technology Review* (May 1977): 20-21, for discussion of neutralization of acid wastes and its cost; (quotatation) John B. Huff, *Furnace Separations for*

- Radioactive Liquid Wastes Report No. PTR-241 (Idaho Falls: 1957, PPCo.), 8.
- See Cyril M. Slansky and John A. McBride, "The Case for Small Reprocessing Plants, *Nucleonics* (September 1962): 43-56.
- 20. Knecht, "Historical Fuel Reprocessing," 40. The ratio of ten gallons of fission product to one million gallons of stored waste was noted in a letter from John B. Huff to Henry Dworshak, August 18, 1958, IHS, Ms. 84, Box 83, File: AEC-Idaho Plant. Geneva Conference *ICPP Report*, p. 534, says lowactivity waste was "discharged directly to the ground." Boron is a neutron "poison," a material that absorbs neutrons and helps prevent accidental criticalities.
- 21. N.J. Rigstad, "ICPP EIS," 1973, p. 7, manuscript provided to author by Jeff Bryant. For a summary of how process chemistry progressed, see Dieter A. Knecht, "Historical Fuel Reprocessing," 35-47. Between 1953 and 1988, the Chem Plant recovered 31,432 kilograms of U-235.
- 22. Don Reid, Cassette Tape # 2, 17-18. See also Rigstad, 12. The fuel cutting facility was at CPP-603, the Fuel Storage Facility. It had to be modified to receive and unload fuel arriving in 75-ton casks, heavier than earlier casks.
- 23. (Dates of RaLa program) U.S. DOE, INEL Historical Dose Evaluation Report No. DOE/ID-12119 (Idaho Falls: DOE/IDO, 1991), Vol. 1, p. A-26. The program was under design and cold testing from March 1954 to November 1956. For a description of the RaLa cell, see Cyril M. Slansky and John A. McBride, "The Case for Small Reprocessing Plants," Nucleonics (September 1962): 44-45; (Operating practices) E.R. Ebersole, "RaLa Monitoring Experience" in A Compendium of Information for Use in Controlling Radiation Emergencies, delivered at a Conference in Idaho Falls on February 23, 1958, HREX Doc. No. 704450; (procedures in early years of RaLa runs prior to addition of filtration equipment) John Horan, interview with Harrop and Shonka, Summer 1994, pp. 2, 4-6; (iodine retention), Phillips Petroleum Company, Idaho Chemical Processing Plant (Idaho Falls: PPCo, n.d., but circa 1965), 33.
- 24. John Byrom, note to author, June 24, 1999. See also Horan, interview with Harrop and Shonka, Summer 1994, pp. 4-5; and Horan, interview with Baldwin and Bacchus, HREX Doc. No. 0726467, pp. 26-27.

Dolts

- John Horan, interview with Burton Baldwin and Thomas Baccus, August 18, 1994, p. 5-6, "redacted version" in HREX Doc. No. 0726467.
- John R. Horan, Annual Progress Report, Health and Safety Division, 1962, Report No. IDO-12033, p. 43, HREX Doc. No. 702992
- 27. INELHistorical Dose Evaluation, iii.

CHAPTER TWELVE

- "Johnson Leaves Position as Idaho AEC Manager, Arco Advertiser, April 30, 1954, 4.
 Also, Jacqui Johnston Batie and William Ginkel, interviews with author, October 14, 1998, and February 3, 1999, respectively.
- (Consolidation) "AEC Contractors Study Merger Move," "Phillips Plans AEC Merger," "Phillips Ends Merger Soon," Post-Register, August 24, August 27, and October 8, 1953, respectively; (employee attitudes) Bernice Paige, April 8, 1999.
- 3. Eisenhower's policy domination is a central theme of Richard G. Hewlett and Jack M. Holl in Atoms for Peace and War 1953-1961 (Berkeley: University of California Press. 1989), xxi; (defense spending) Harry S. Truman, Memoirs, Years of Trial and Hope, Volume 2 (Garden City, N.Y.: Doubleday and Co., 1956), 306-312; (quotation) Dwight David Eisenhower, Mandate for Change, 1953-1956 (Garden City, N.Y.: Doubleday and Co., 1965), 452; (massive retaliation) Chester J. Pach, Jr., and Elmo Richardson, The Presidency of Dwight David Eisenhower (Lawrence: University Press of Kansas, 1991), 76-81. The Army's share of the budget shrank from 33 percent to 25 percent, and the Air Force's grew from 39 percent to 47 percent.
- (Scare the country, Atoms for Peace quotations, Operation Candor) Herbert S. Parmet, Eisenhower and the American Crusades (New York: Macmillan Company, 1972), 386; (national objectives) U.S. AEC, Major Activities in the Atomic Energy Programs, January—June 1953 U.S. Atomic Energy Commission (Washington, D.C.: U.S. AEC, 1953), 18.
- 5. (Quotation) Hewlett, New World, 5.
- Public Law 83-703 was adopted by the Second Session of the 83rd Congress and became effective August 30, 1954.

- J.W. Henscheid, interview with author, December 10, 1998.
- J.R. Huffman, MTR Technical Quarterly Report, Second Quarter 1954, Report No. IDO-16191 (Idaho Falls: PPCo, 1954), 17; and Huffman, MTR Technical Quarterly Report, Third Quarter 1954, Report No. IDO-209, 12.
- 9. Deslonde deBoisblanc, interview with author, January 16, 1999.
- 10. R.L. Heath, Scintillation Spectrometry,
 Gamma-Ray Spectrum Catalogue, 2nd
 edition, Report No. IDO-16880-1 (Idaho
 Falls: PPCo, 1964). Report number for the
 first edition was IDO-16408. According to
 D.R. deBoisblanc, Heath "pushed the
 resolution by more than a factor of sixteen
 using lithium-drifted germanium and silicon,
 plus low noise amplifiers employing liquid
 nitrogen-cooled field effect transistors. He
 also introduced the use of digital computers
 to reduce the data."
- 11. 1965 Thumbnail Sketch (Idaho Falls: Idaho Operations Office, 1965), 17; deBoisblanc, July 14, 1999.
- (Hydraulic rabbit) US AEC, Major Activities in the Atomic Energy Program July—
 December 1953. (Washington, D.C.: US AEC, January 1954), 25; (Argonne loop)
 L.W. Fromm, Design Evaluation of In Reactor Tube for Argonne Water Loop at
 MTR, Report No. ANL-5403 (Lemont,
 Illinois: Argonne National Laboratory,
 1955), 5.
- W.E. Nyer, et al. Proposal for a Reactivity Measurement Facility at the MTR, Report No. IDO-16108 (Idaho Falls: PPCo, 1953), 6-8.
- 14. J.W. Henscheid, personal communication to author, December 10, 1998.
- J.R. Huffman, MTR Technical Branch Quarterly Report for First Quarter 1955, Report No. IDO-16229 (Idaho Falls: PPCo, 1955), 24.
- Pamphlet, Gamma Irradiation Facility, A Fact Sheet, no author, p. 3-5, found attached to 1957 Thumbnail Sketch.
- 17. Walter C. Sparks, interview published in Susan M. Stacy, ed., *Conversations, A Companion Book to Idaho Public Television's "Proceeding On ThroughA Beautiful Country)* (Boise: Idaho Educational Public Broadcasting Foundation, 1990), 266-267.

- Walter C. Sparks, interview with author, May 13, 1999.
- Sparks, May 13, 1999; George A. Freund, Suitability of Potato Products Prepared from Irradiated and Chemically Inhibited Potatoes and Freund, Current Status and Potential of Irradiation to Prevent Potato Sprouting, Reports No. IDO-10042 and IDO-11300-Addendum, respectively, both (Idaho Falls: Western Nuclear Corporation in cooperation with Engineering Committee, Potato Processors of Idaho, February 1965); and "Commercial Interest Increasing Rapidly in Food Irradiation, Nucleonics (December 1963): 23.
- (40 megawatts) IDO-16254, 6; (questions) see series of Phillips Technical Branch quarterly reports for 1955 through 1957.
- W.P. Connor, MTR Technical Branch Quarterly Report, Report No. IDO-16297 (Idaho Falls: PPCo, 1956), 5.
- 22. "Test Reactors—The Larger View," *Nucleonics* (March 1957): 55.
- 23. Philip D. Bush, "ETR: More Space for Radiation Tests," *Nucleonics* (March 1957): 41-42. The extra depth required for the control rods meant that a portion of the foundation had to be blasted through lava rock. See also R.M. Jones, *An Engineering Test Reactor for the MTR Site (A Preliminary Study)*, Report No. IDO-16197 (Idaho Falls: PPCo, 1954), 7.
- 24. 1965 Thumbnail Sketch, 14; J.W. Henscheid, December 10, 1999. For more information on the ETR, see "ETR: More Space for Radiation Tests," Nucleonics (March 1957): 41-56.

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- Horan, Baldwin and Baccus 1994 interview, HREX Doc. No. 0726467. Bracketed inserts within quotation are from *INELHistorical Dose Evaluation*, Volume 2, pp. A-180 to A-182.
- (Johnston announcement) "AEC to Add Aircraft Unit Here," Post-Register, July 29, 1952, 1; See "AEC Announces Preliminary Plans for Airplane Reactor Will be Ready Around November 15," Arco Advertiser, November 7, 1952, 1; "AEC Sets January 30 as Bid Opening Date for \$6,000,000 Facilities at Aircraft Reactor Site," Post-Register (assumed), January 3, 1953, 1; "Utah Construction Low Bidder on AEC Aircraft Reactor," Arco Advertiser, February

- 20, 1953; "Contact Let for AEC Aircraft Propulsion Reactor," *Arco Advertiser*, March 27, 1953, 1; "AEC Announced Bids for the Construction of an Access Road to the Prototype Aircraft Propulsion Reactor," *Post-Register*, August 31, 1952.
- "Allan C. Johnson Gets Idaho AEC Post; Bowden Goes to FOA," Post-Register, May 13, 1954, 1; "Allan Johnson Named New AEC Manager, Arco Advertiser, May 14, 1954, 1.
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- 5. Hewlett, Atomic Shield, 71-74, 106.
- John Tierney, "Take the A-Plane: The \$1
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 also Henry W. Lambright, Shooting Down
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- To Senator William F. Knowland from Henry Dworshak, November 24, 1956; IHS Dworshak Papers, Box 59, File: Legis AEC Miscellaneous.
- 8. Among the public relations documents produced by General Electric in 1959 was *Idaho Test Station, Idaho Falls, Idaho*, a pamphlet describing GE's activities concerning the ANP program. It includes explanations and diagrams explaining the direct cycle concept. Copy in Dworshak Papers, Box 112 File: AEC Idaho Plant. Pratt and Whitney contracted to develop the indirect cycle, but got a late start due to contract disputes.
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- 12. For description of HTREs, see G. Thornton, A.J. Rothstein, and D.H. Culver, ed. Comprehensive Technical Report, General Electric Direct-Air-Cycle Aircraft Nuclear Propulsion Program, Program Summary and References, Report No. APEX-901 (Cincinatti: GE ANP Department, Atomic Products Division, 1962),
- See also news release, "General Electric ANPDepartment Completes Testing of Heat Transfer Experiment No. 3," issued by AEC IDO, January 27, 1961, Dworshak Papers, Box 122-B, File: Atomic Energy Commission, Idaho Press Releases.
- General Electric, ANPP Engineering Program Progress Report No. 18, Report No. APEX-18 (Cincinatti: GE ANP, 1955), 7; APEX-901, 37; Tierney, "Take the A-Plane," 50.
- 14. (Shield weight) Tierney, "Take the A-Plane," 50; APEX-18, 195.
- 15. Gamertsfelder, "Oral History," Report No. DOE/EH-0467.
- 16. Comptroller General of the United States, Review of Manned Aircraft Nuclear Propulsion Program, Atomic Energy Commission and Department of Defense, Report to the Congress of the United States (Washington, D.C.: General Accounting Office, 1963), 138. Congress authorized construction of the FET(hangar), support buildings, and a runway in July 1955. Detailed design commenced in March 1956, and groundbreaking occurred in September 1957. The facility was ready in July 1959. Hereafter, Comptroller General.
- 17. Richard Meservey, interview with author, October 12, 1998.
- 18. AEC/IDO news release, "General Electric ANPDepartment Completes Testing of Heat Transfer Experiment No. 3," January 27, 1961, IHS, Dworshak Papers, Box 122-B, File: Atomic Energy Commission, Idaho Press Releases. See also "HTRE 3 Completes 120 Hours of Operation," GE News, Idaho Test Station, Aircraft Nuclear Propulsion Department (April 1, 1960): 1. Issue found in IHS, Dworshak Papers, Box 112, File: AEC Idaho Plant; and APEX-901, p. 50, 57.
- John Horan, interview with Baldwin and Baccus, HREX Doc. No. 0726467, p. 7. See also Gamertsfelder, DOE/EH-0467.
- 20. APEX-18, 196.

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- 22. Gantz, Nuclear Flight, 175, 178.
- D.J. Blevins, et al. Flight Engine Test Facility Design Criteria, Report No. APEX-225 (Idaho: GE ANPDepartment Idaho Test Station, 1955), 15-21.
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 The acronym comes from the letters
 BOiling water Re Actor eXperiment.
- Holl, Argonne, 118; Andrew W. Kramer, Understanding the Nuclear Reactor (Barrington, Illinois: Technical Publishing Co., 1970), 37, 70.
- 3. J.R. Dietrich and D.C. Laymans, *Transient* and Steady State Characteristics of a Boiling Reactor: The Borax Experiments, 1953, Report No. ANL-5211, February 1954; Holl, Argonne, 118; Plastino, Coming of Age, quoting Ray Haroldsen, 64.
- 4. Holl, Argonne, 118.
- 5. Holl, Argonne, 119.
- Holl, Argonne, 120; (talk of dynamite), Clyde Hammond, interview with author, October 15, 1998.
- 7. Plastino, *Coming of Age*, quoting Haroldsen, 64
- 8. For account of 1955 Geneva Conference, see Laura Fermi, Atoms for the World: United States Participation in the Conference on the Peaceful Uses of Atomic Energy (Chicago: University of Chicago Press, 1974).
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 core made of a fuel containing Thorium
 Oxide and Uranium Oxide.
- "Arco First City in United States Lighted by Atomic Power," Arco Advertiser, August 12, 1955, 1; "Arco Gets Premier Showing of AEC Film, Arco Advertiser, August 19, 1955, 1; Plastino, Coming of Age, 65.
- 11. (Borax tests) Argonne National Laboratory, Annual Report for 1959, ANLReport No. ANL-6125 (Chicago: ANL, 1959), 94. See also J.O. Roberts, "Selected Operating Experience of Commission Power Reactors," a paper presented to the American Institute of Electrical Engineers, June 18, 1961, in IHS, Ms 84, Box 122 B, File: AEC-Press Releases; (EBWR) Holl, Argonne, 150; Robert L. Loftness, Nuclear Power Plants: Design, Operating

- Experience, and Economics, (Princeton, N.J.: Van Nostrand (Nuclear Science Series), 1964), 167-213; to Senator Dworshak from Francis K. McCune, GE, September 25, 1958; IHS, Ms 84, Box 104, File: Legislative AEC Misc 1959.
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- 13. Richard L. Doan, "Two Decades of Reactor Safety Evaluation," Memorial Lecture in honor of Dr. C. Rogers McCullough prepared for Winter Meeting of the American Nuclear Society, Washington, D.C., November 15-18, 1970, p. 3. For general histories of reactor safety, see George T. Mazuzan and J. Samuel Walker, Controlling the Atom (Berkeley: University of California Press, 1984) and J. Samuel Walker, Containing the Atom (Berkeley: University of California Press, 1992).
- Jack M. Holl, et al., United States Civilian Nuclear Power Policy, 1954-1984: A History (Washington, D.C., U.S. DOE, 1985), 4.
- 15. Warren Nyer, interview with author, February 1, 1999.
- (Location) T.R. Wilson An Engineering Description of the SPERT-1 Reactor Facility, Report No. IDO 16318 (Idaho Falls: ANL, 1957), 8. (vocabulary) to Glenn Seaborg from Senator Dworshak, November 16, 1961, IHS MS 84, Box 122 B, File: AEC-Idaho Plant.
- "SPERT-2 Features Versatility," Nucleonics (June 1960): 120; pamphlet, "Special Power Excursion Reactor Tests" (Idaho Falls: Phillips Petroleum Company, n.d.), 18.
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- "Special Power Excursion Reactor Tests," (Idaho Falls: Phillips Petroleum Company. n.d.); Warren Nyer, February 1, 1999.
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- 21. Holl, Argonne, 115-116.
- 22. J.O. Roberts, "Selected Operating Experience," 37.
- 23. Holl, Argonne, 141-143.
- 24. "AEC Tells How Reactor Failed," Business Week (April 14, 1956): 34; Jerome D. Luntz, "Nuclear Accidents Are Everybody's Business, Nucleonics (May 1956): 39.

- Argonne National Laboratory, Annual Report for 1957, Report No. ANL-5870 (Chicago: ANL, 1958), 83.
- Loftness, Nuclear Power Plants, 339; ANL-West, EBR-II since 1964 (Idaho Falls: ANL-W, 1983).
- For design and other details, see George Freund, et al, *Design Summary Report on* the Transient Reactor Test Facility, TREAT, Report No. ANL-6034 (Chicago: ANL, 1960).
- ANL, Annual Report 1959, 102; Edmund S. Sowa, "First TREAT Results—Meltdown Tests of EBR-II Fuel," Nucleonics (June 1960): 122-124.
- These were the Argonne Fast Source Reactor (AFSR) and Zero-Power Reactor Three (ZPR-III). ZPR-I and -II were not located at the NRTS.
- "About the Idaho Division..." (Idaho Falls: Argonne Idaho Facilities, 1964), Fact Sheet EBR-II, page 1-2. The approach to designlevel power production was gradual, attained on September 25, 1969.

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Epigraph: Interview of George Voelz, "Remembering the Early Years," Report No. DOE/EM 0454, November 29, 1994, HREX Doc. No. 0727845.

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- For a history of the Army Reactor Program at the NRTS, see Susan M. Stacy, Historical American Engineering Record, Idaho National Engineering Laboratory, Army Reactor Experimental Area, HAER No. ID-33-D.
- 3. Suid, Army's Nuclear Program, 83.
- See "Description of ALPR," Nucleonics (February 1958), and pull-out section following p. 54.
- Argonne had named the reactor "Argonne Low Power Reactor," or ALPR.

- 6. The GCRE was the eighth reactor type developed by the AEC Nuclear Reactor development program, selected for both military and civilian potential. The name of the AREAreactor complex was changed later to ARA(Army Reactor Area) and still later to ARA(Auxiliary Reactor Area). U.S. AEC press release, June 6, 1956; IHS Dworshak Papers, Mss 84, Box 55, File: AEC—Idaho Plant.
- Norman Engineering Co., Master Plan Study for the Army Reactor Experimental Area, Report No. IDO-24033 (Idaho Falls: Norman Engineering, 1959), Section II (no p.n.).
- 8. R.T. Canfield, *SL-1 Annual Operating Report, February 1959-February 1960*, Report No. IDO 19012 or CEND-82 (Idaho Falls: Combustion Engineering, 1960), 7. SL-1 was the first power plant reactor in the country to use aluminum-clad fuel elements, which heretofore had been used only in test reactors like the MTR. A new alloy overcame the low melting point of aluminum, and after SL-1, aluminum alloys were used widely. See Suid, *Army's Nuclear Program*, 83.
- "SL-1 Explosion Kills 3; Cause and Significance Unclear," *Nucleonics* (February 1961): 17.
- 10. See drawing in *Nucleonics*, (February 1969): 54 ff. The fuel zone was 27.8 inches long.
- (Water hammer) General Electric, Final Report of SL-1 Recovery Operation (Idaho Falls: GE Report No. IDO-19311, 1962), p. I-6.
- 12. Staff of the JCAE, "Summary of the SL-1 Reactor Incident at the National Reactor Testing Station in Idaho on January 3, 1961," January 10, 1961, IHS, Dworshak Papers, Box 122 B, File: AEC Id Press Releases. Other details of the accident were reported in *Nucleonics* (February 1961): 17-23; and in a series of press releases from the Idaho Operations Office after the accident, copies in Dworshak papers.
- 13. "Sequence of Events Related to the SL-1 Accident at the National Reactor Testing Station, Idaho, on January 3, 1961." HREX document #0715561. No author or date. This report was an appendix to an unidentified AEC document. See also *Nucleonics* (February 1961); and "Detailed Events Chain at SL-1 Reactor Told," *Post-Register*, January 12, 1961.

- 14. A Class II event would impact other facilities on the site; a Class III, territory beyond the boundaries of the Site. A road block was not required. C. Wayne Bills recalled that the HPs may have stopped one or two autos in order to determine whether their tires had picked up any contamination.
- SL-1 Report Task Force, IDO Report on the Nuclear Incident at the SL-1 Reactor on January 3, 1961, at the National Reactor Testing Station, Report No. IDO-19302 (Idaho Falls: Idaho Operations Office, 1962), 54.
- "Big Hazard at Idaho Atom Site? Traffic Accidents, Says Nurse," *Post-Register*, February 23, 1961.
- Dr. George Voelz, "Remembering the Early Years," Oral History Interview, November 29, 1994, HREX Doc. No. 0727845.
- 18. The men were Specialist 5 John A. Byrnes, 27, U.S. Army; Specialist 4 Richard L. McKinley, 22, U.S. Army; and Construction Electrician Richard C. Legg, 26, U.S. Navy. Byrnes had been certified at SL-1 in February 1960; Legg, in September 1960. McKinley was a trainee.
- John Horan, interview with author, July 16, 1998.
- 20. Horan, July 16, 1998.
- 21. "Findings of the Board of Investigation," *Nuclear News* (July 1961): 15.
- 22. INELHistoric Dose Evaluation, Volume 2, p. A-83 to A-85. See also Volume I map, p.27.
- 23. (Squad rescue) Staff of the JCAE, "Summary of the SL-1 Reactor Incident," 2-3; (GCRE decontamination, logbook, neutron detector) film "The SL-1 Accident," (Idaho Falls: Idaho Operations Office).
- 24. The General Manager's Board of Investigation, *Report on the SL-1 Incident, January 3, 1961*, Press Release No. 61-24 (SL-1), (Idaho Falls: Idaho Operations Office, June 11, 1961), 31.
- (Team recovery) IDO Press release, January
 1961, Dworshak Papers, box 122 B;
 (Dugway, mockup, procedures) film "The SL-1 Accident."
- George Voelz, M.D., November 29, 1994;
 John Horan and Wayne Bills, "What Have We Learned? Health Physics at SL-1," Nucleonics (December 1961): 48.
- 27. (Luedecke proposal) John Horan, July 16, 1998; (caskets) "Blast Victim's [sic] Rites

- Await AEC Action," Salt Lake Tribune,
 January 17, 1961; (lead wrap) C. Wayne
 Bills, February 3, 1999; "Military Gets
 Bodies of NRTS Victims," Post-Register,
 January 23, 1961; "Atom Victim is Buried in
 Lead Vault," Utica Observer-Dispatch,
 January 25, 1961; "ACatastrophe New to
 Man," New York Herald Tribune, January 29,
 1961. The AEC required that the cemeteries
 accepting the caskets must have perpetual
 care, that the graves not be disturbed without
 AEC permission, and that the graves be
 shielded with concrete and dirt.
- Jay Kunze, February 5, 1999. The AEC contracted General Electric to disassemble and analyze the reactor on May 5, 1961; (Dugway) C.W. Bills, personal communication, June 25, 1999.
- 29. Kunze, February 5 and July 6, 1999.
- 30. Kunze, February 5, 1999.
- (Crane) Wayne Bills, interview with author, February 3, 1999; (vessel in transit) General Electric, *Final Report of SL-1* p. II-37, II-39.
 See also "SL-1 Reactor Core Trucked to Hot Cell for Study," *Arco Advertiser*, December 8, 1961, 3.
- 32. General Electric Final Report, ii.

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- 1. (Monster quote) Robert F. Alkire,
 "'Monsters'Huddle in Weird Stillness,"
 Associated Press story published in Salt
 Lake Tribune, January 8, 1961, and as
 "Moon-like Eastern Idaho Desert Scene of
 Nuclear 'Monster' Runaway," in Idaho
 Daily Statesman, same date, 5; (sleeping
 quote) "A-Reactors Won't Blow Us Up,"
 The Deseret News and Telegram, January 6,
 1961.
- (Mishaps) "NRTS Marks First Fatal Atomic Mishap," *Idaho Daily Statesman*, January 5, 1961; (instructed) Frank Carey, "Chemical Explosion in Nuclear Reactor Potentially More Perilous than A-`Blast,' *Idaho Daily Statesman*, January 5, 1961.
- The IDO compiled 154 press clippings from around the country in a document "The SL-1 Accident, Press Clippings." A copy is extant at the IDO public information office in Idaho Falls, Idaho.
- 4. (Siting) John W. Finney, "Blast Stresses A-Question," *Salt Lake Tribune*, January 5, 1961; (set back) "What Happened in

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- Nuclear Blast," *Business Week*, January 7, 1961.
- (Reuther quote) "Press Release from Industrial Union Department of AFL-CIO, January 5, 1961, at IHS, Ms 84, Box 122 B, File: "AEC-Idaho Plant;" (defeat) "AMonth After the Idaho Accident," Oak Ridger, February 6, 1961.
 - The forty-item list included incidents such as the MTR fuel rod with a pin-hole leak (described in Chapter 12) and similar events that had occurred in other AEC facilities across the country. The information was available for review in unclassified AEC literature.
- To Senator Henry Dworshak from Donald E. Seifert and George Dragich, May 11, 1061, in IHS, Mss 84, Box 122 B, File: AEC-Idaho Plant.
- To Senator Dworshak from John T. Conway, June 20, 1961, IHS, Mss 84, Box 122 B, File: AEC Miscellaneous.
- (Wilson) To Senator Dworshak from John T. Conway, acting executive director of JCAE, June 1, 1961, in IHS, Mss 84, Box 122 B, File: AEC-Idaho Plant.
- 9. Atomic Energy Commission Investigation Board, *SL-1 Accident* (Washington, D.C.: Joint Committee Print, U.S. Government Printing Office, 1961), ix.
- "Johnson Resigns at AEC," Arco Advertiser," December 8, 1961, 1; John Horan, C. Wayne Bills interviews with author, July 16, 1998, and February 2, 1999, respectively.
- 11. To Ramey from A.R. Luedecke, January 12, 1961, in IHS, Mss 84, Box 122 B, File: AEC-Idaho Press Releases; "AEC Surveys Safety of Licensed Reactors," *Post-Register*, January 13, 1961; "Safety Survey Launched for all Reactors Licensed by AEC," *Idaho Daily Statesman*, January 13, 1961; "AEC Asks U. Bolster Reactor Safety," *Salt Lake Tribune*, January 13, 1961.
- 12. J.W. Henscheid, interview with author, December 10, 1998.
- 13. David Okrent, On the History of the Evolution of Light Water Reactor Safety in the United States (Los Angeles: University of Los Angeles, 1978), pp. 6-102 to 6-106. Okrent reproduces a letter from the ACRS to A.R. Luedecke, December 31, 1962.
- 14. (Greenland) To Ramey from Luedecke, January 12, 1961, op cit.; (ML-1)

- AEC/Idaho Operations press release, February 11, 1961, IHS, Dworshak Papers, Box 122 B, File "AEC—Press Releases." Also, Hogerton, *Atomic Energy Desk Book*, 33. The Army operated SM-1 at Fort Belvoir; SM-1Aat Fort Greeley, Alaska; PM-2Aat Camp Century, Greenland; PM-1 at Sundance Air Force Base, Wyoming; PM-3Aat McMurdo Sound, Antarctica; PL-3 at Byrd Station; and *Sturgis*, a barge.
- 15. (Power dates) Suid, *Army's Nuclear Program*, 91.
- T.L. Murphy, Final Disassembly and Examination of the ML-1 Reactor Core, IDO-171-90, (Idaho Falls: PPCo, 1966), 110-111; "Economic Military Power Arrives, But Pentagon Hesitates," Nucleonics (April 1960): 27; Suid, Army's Military Reactors, 92-93. See also Stacy, Army Reactor Experimental Area, HAER No. ID-33-D.
- 17. The SL-1 Burial Ground was regraded and recontoured in 1997 after a 1996 "Record of Decision" signed by the Department of Energy, the Environmental Protection Agency, and the State of Idaho. The new natural-material "cap" was to improve the isolation of hazardous materials from humans and the surrounding environment. Erik Simpson, "Agencies agree to cap reactor burial grounds," INELNews (February 6, 1996): 7.
- 18. "Reactor Safety: Accident Control Study," Nucleonics (April 1964): 28. See John R. Horan and C. Wayne Bills, "What Have We Learned? Health Physics at SL-1," Nucleonics (December 1961): 48; Voelz, November 29, 1994. The film, "The SL-1 Accident," produced by Idaho Operations Office circa 1962, was in three parts, Phase 1, 2, and 3. All were converted to video format.
- 19. C. Wayne Bills, personal communication to author, June 25, 1999.
- (Exposures) IDO Report on the Nuclear Incident at the SL-1, 27; John R. Horan and C. Wayne Bills, "What Have We Learned? Health Physics at SL-1," Nucleonics (December 1961): 43.
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DOLES

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Acronyms

630-A	High Temperature Marine Propulsion Reactor	EBR	Experimental Breeder Reactor
710	Fast Spectrum Refractory Metals Reactor		Experimental Beryllium Oxide Reactor
A1W	Aircraft carrier, first prototype, Westinghouse	EBWR	Experimental Boiling Water Reactor
	(Also known as the Large Ship Reactor A and B)	ECCS	
A/E	architect/engineering	ECF	
ACRS	Advisory Committee on Reactor Safeguards	EINIC	East Idaho Nuclear Industry Council
AEC	Atomic Energy Commission	EIS	Environmental Impact Statement
AFSR	Argonne Fast Source Reactor	EOCR	Experimental Organic Cooled Reactor
AI	Atomics International	EPA	Environmental Protection Agency
ANC	Aerojet Nuclear Corporation	ERDA	Energy Research and Development Administration
ANP	Aircraft Nuclear Propulsion	EROB	Engineering Research Office Building
ARA	Army Reactor Area, (Later Auxiliary Reactor Area)	ETR	Engineering Test Reactor
AREA	Army Reactor Experimental Area	ETRC	Engineering Test Reactor Critical Facility
ARMF	Advanced Reactivity Measurement Facility	FARET	Fast Reactor Test Facility
ATR	Advanced Test Reactor	FCF	Fuel Cycle Facility
ATRC	Advanced Test Reactor Critical Facility	FET	Flight Engine Test
ATWS	Anticipated Transients Without Scrams	FFTF	Fast-Flux Test Facility
BORAX	Boiling Water Reactor Experiment	FLECHT	Full-length Emergency Core Heating Tests
BPA	Bonneville Power Administration	FPC	Federal Power Commission
CE	Combustion Engineering	FRAN	Nuclear Effects Reactor
CERCLA	Comprehensive Environmental Response,	FRAP	Fuel Rod Analysis Program
	Compensation, and Liability Act	FWPCA	Federal Water Pollution Control Administration
CERT	Controlled Environmental Radioiodine Tests	GCRE	Gas Cooled Reactor Experiment
CET	Critical Experiment Tank	GE	General Electric
CFA	Central Facilities Area	HOTCE	Hot Critical Experiment
CFRMF	Coupled Fast Reactivity Measurement Facility	HP	health physicist
CP-1	Chicago Pile Number One	HTRE	Heat Transfer Reactor Experiment
CPP	Chemical Processing Plant	ICPP	Idaho Chemical Processing Plant
CRADA	Cooperative Research and Development Agreement	IDO	Idaho Operations Office
CRCE	Cavity Reactor Critical Experiment	IDWR	Idaho Department of Water Resources
DEW	Distant Early Warning System	IET	Initial Engine Test
DoD	Department of Defense	IFR	Integral Fast Reactor
DOE	Department of Energy	INC	Idaho Nuclear Corporation
DOE-ID	Department of Energy-Idaho Operations Office	INEC	Idaho Nuclear Energy Commission

INEL	Idaho National Engineering Laboratory	S5G	submarine reactor, 5th prototype, General Electric
INEEL	Idaho National Engineering and Environmental		(Also known as the Natural Circulation Reactor)
	Laboratory	SCRCE	Spherical Cavity Reactor Critical Experiment
INFCE	International Nuclear Fuel Cycle Evaluation	SDI	Strategic Defense Initiative
INTEC	Idaho Nuclear Technology and Engineering	SIS	Special Isotope Separations
	Center	SL-1	Stationary Low-Power Reactor
IRC	INEEL Research Center	SM-1	Stationary Medium-Power Reactor
ISC	INEEL Supercomputing Center	SMC	Specific Manufacturing Capability
ISU	,	SNAP	Systems for Nuclear Auxiliary Power
JCAE	Joint Committee on Atomic Energy	SPERT	Special Power Excursion Reactor Test
KAPL	Knolls Atomic Power Laboratory	STEP	Safety Test Engineering Program
LMFBR	Liquid Metal Fast Breeder Reactor	STR	Submarine Thermal Reactor
LOCA	Loss-of-Coolant Accident		(Also known as S1W or, Submarine reactor, 1st
LOFT	Loss of Fluid Test		prototype, Westinghouse)
MIT	Massachusetts Institute of Technology	STR	Split Table Reactor
ML-1	Mobile Low-Power Reactor	SUSIE	Shield Test Pool Facility
MTR	Materials Testing Reactor	TAN	Test Area North
NaK	eutectic alloy of sodium (Na) potassium (K)	THRITS	Thermal Idaho Reactor Test Station
NAS	National Academy of Science	TMI	Three Mile Island
NASA	National Aeronautic and Space Administration	TRA	Test Reactor Area
NEPA	National Environmental Policy Act	TSA/B	Technical Support Building
NERI	National Energy Research Initiative	TREAT	Transient Reactor Test
NERP	National Environmental Research Park	TRU	transuranic
NOAA	National Oceanic and Atmospheric Administration	U of I	University of Idaho
NPG	Naval Proving Ground	UAW	United Auto Workers
NPR	New Production Reactor	UP&L	Utah Power and Light
NRAD	Neutron Radiography Facility	USGS	United States Geological Survey
NRC	Nuclear Regulatory Commission	USSR	Union of Soviet Socialist Republics
NRF	Naval Reactors Facility	WAG	Waste Area Group
NRTS	National Reactor Testing Station	WBRR	Western Beam Research Reactor
NSF	National Science Foundation	WCB	Willow Creek Building
NWCF	New Waste Calcining Facility	WCF	Waste Calcining Facility
OAC	operating area confinement	WERF	Waste Experimental Reduction Facility
OMRE	Organic Moderated Reactor Experiment	WIPP	Waste Isolation Pilot Plant
OU	operable unit	WOW	Woman Ordnance Worker
PBF	Power Burst Facility	ZPR	Zero Power Reactor
RAF	Remote Analytical Facility	ZPPR	Zero Power Physics Reactor
RaLa	radioactive lanthanum		(Previously known as the Zero Power Plutonium
RIA	reactivity-initiated accidents		Reactor)
REM	roentgen equivalent man		

RMF Reactivity Measurement Facility
RWMC Radioactive Waste Management Complex

Glossary

Activation product

Upon bombardment with neutrons, some materials absorb neutrons into their nuclei, forming new and usually radioactive isotopes.

Alpha particle

A positively charged nuclear particle identical with the nucleus of a helium atom. It consists of two protons and two neutrons. Alpha particles can be stopped by a sheet of paper.

Anti-Cs

Slang speech meaning "anti-contamination," referring to special clothing worn by people requiring protection from radiation.

Atom

The smallest particle of an element that can exist either alone or in combination with other elements. Atoms are made up of electrons, neutrons, and protons.

Atomic energy

Energy that can be liberated by changes in the nucleus of an atom, such as by fission or fusion. Contemporary scientists prefer to use the term "nuclear energy."

Atomic number

A characteristic of an element, the number of protons in the nucleus.

Aquifer

A water-bearing stratum of permeable rock, sand, or gravel.

Background radiation

The radiation in an ambient environment. It includes cosmic rays from outer space, radon gas, and other forms of radiation from natural sources (such as granite) and human-made sources (dental X-rays, fall-out from nuclear explosions).

Beta particle

An electron or positron ejected from the nucleus of an atom during radioactive decay. The mass of an electron is equal to 1/1837 that of a proton. It can be stopped by an inch of wood or a thin sheet of aluminum.

Bin set

A cluster of storage containers at the Idaho Nuclear Technology Engineering Center built to store solid calcine waste. The waste is highly radioactive, and the bin sets are heavily shielded.

Blowdown

A term used to describe sudden depressurization upon the breaking of a pipe carrying pressurized water.

Boiling water reactor

A nuclear reactor concept in which the coolant, water, is permitted to boil as it absorbs the heat of the nuclear reaction. The resultant steam drives a turbine and generates electricity.

Breeder reactor

A nuclear reactor concept in which the operation produces a net increase in fissionable material. That is, more fissionable material is produced than is consumed.

Calcine

As a noun, the dry solid (grainy or granular) product of a chemical process removing liquids from a solution. As a verb, the heating of a material at a high temperature to drive off volatile materials.

Cerenkov radiation

A blue-white light produced when gamma rays hit electrons in water. The energy of the gamma rays is sufficiently great that the electrons move through the water faster than light moves through water.

Cesium-137

A radioactive isotope of the element cesium, which emits gamma radiation. It is an important fission product.

Chain reaction

A self-sustaining sequence of events occurring when a neutron splits a fissionable atom (of uranium, for example) and releases sufficient neutrons to cause other atoms to split in the same way.

China Syndrome

A figure of speech referring to a theoretical melting of nuclear reactor fuel which would occur upon a loss of coolant to the fuel. The fuel would melt, penetrate the reactor vessel, drop to the concrete floor of the building, and reach the soil below. The phrase comes from the expression "dig a hole all the way to China," a fantasy of (American) children who believe China to be on the opposite side of the globe from their own playground.

Chicago Pile-1

The name of the first nuclear reactor to go critical, so called because graphite blocks were piled upon each other to construct the reactor.

Cladding

The outer layer of metal over the fissionable material in a nuclear fuel element, typically aluminum or zirconium.

Cladding promotes the transfer of heat from the fuel to the coolant and contains fission products and activation products within the fuel element.

Cold run

A test of a chemical process and equipment using non-radioactive materials.

Cold shutdown

A reactor condition in which the coolant temperature has been reduced to 200° F or below, the pressure has been reduced to atmospheric pressure, and the chain reaction has stopped.

Cold War

A conflict over ideological differences between the United States and the Soviet Union and their respective allies lasting from the late 1940s until early in the 1990s. It was carried on by means other than sustained or direct military action.

Containment building

A safety feature of most commercial nuclear reactor power plants. The airtight building, typically engineered to contain gases and pressures that might be released in an accident, houses the reactor, pressurizer, coolant pumps, and other equipment.

Control rod

A device within a nuclear reactor made of materials which absorb neutrons. Control rods help dampen or permit the reactor's chain reaction.

Contamination, radioactive

Unintentional or undesirable contact of a person, object, or material with radioactive substances.

Control room

The operating center of a nuclear reactor from which the reactor is operated and monitored.

Coolant

In a nuclear reactor, a gas or fluid (such as water or liquid metal) sent past the fuel elements to collect and carry away the heat generated by the nuclear reaction.

Core

That part of the nuclear reactor consisting of the fuel and control elements, the coolant, and the vessel containing these.

Criticality

The point at which a nuclear reactor is just capable of sustaining a chain reaction.

Critical mass

The minimum amount of nuclear fuel necessary to sustain a chain reaction.

Curie

A measure of radioactivity, a curie is that quantity of material that decays at a rate of 3.7×10^{10} disintegrations per second.

D&D

An abbreviation for "decontamination and decommissioning," particularly of a building or structure that once housed active nuclear activities and may have been contaminated in the process. Historic uses of the term may also have referred to "dismantling" or "demolishing."

Decontaminate

A process removing radioactive materials from a person, place, or object.

Decay

The spontaneous ejection of particles by radioactive materials. Synonym for radioactive disintegration.

Depleted uranium

Uranium that, through the process of enrichment, has been stripped of most of the uranium-235 it once contained. It has more uranium-238 than natural uranium, but is referred to as "depleted."

Dose

A specific amount of ionizing radiation or a toxic substance absorbed by a living being.

GLOSSARY

Dosimeter

A device such as a film badge which can be worn by a person (or placed somewhere in the environment) and is used to measure the radiation dose received over a period of time.

Electron

An elementary particle consisting of a charge of negative energy. Electrons are said to circle the nucleus of an atom.

Emergency Core Cooling System

An emergency backup system designed to inject cooling water into the core of a reactor in the event that the normal cooling system fails. This safety requirement is intended to prevent the overheating of the fuel and subsequent melting.

Enriched uranium

Uranium which has been modified from its natural state to contain a higher concentration of the isotope uranium-235 than natural uranium.

Excursion

A term used to describe an unexpected or accidental increase in the power level of a nuclear reaction.

Fallout

Radioactive particles and gases resulting from a nuclear explosion which gradually descend to earth.

Film badge

A piece of masked photographic film worn by nuclear workers. The film is darkened by radiation and can be analyzed to indicate how much exposure the film and the badge wearer received over a period of time.

Fission

The splitting of an atomic nucleus resulting in the creation of lighter elements, heat, free neutrons, and other particles.

Fission product

Any of several lighter elements or particles created by the nuclear fission of a heavy element such as uranium.

Flux

The flow or stream of neutrons emanating from nuclear fission.

Fossil fuels

Coal, oil, and natural gas are referred to as "fossil" fuels because they are the remains of plants and animals that lived on earth millions of years ago.

Fuel cycle

The life cycle of a fuel including the complete sequence of steps beginning with mining and refining an ore and ending with the disposition of the waste products after the fuel has been beneficially used.

Fuel reprocessing

A chemical process, usually involving several steps, that recovers uranium-235 and other fissionable products from spent fuel.

Fuel assembly

An arrangement of nuclear fuel and its cladding material into a particular form and shape for use in a nuclear reactor. Fuel may be assembled in plates, rods of various diameters, or other shapes.

Fusion

The union of atomic nuclei to form heavier nuclei resulting in the release of enormous quantities of energy. The process usually requires conditions of extreme heat and pressure.

Gamma radiation

High-energy, high penetrating electromagnetic radiation emitted in the radioactive decay of many radionuclides. They are similar to X-rays.

Geiger counter

An instrument used to detect and measure beta and gamma radiation.

Half-life

The time it takes for one-half of any given number of unstable atoms to decay (disintegrate). Half-life is unaffected by temperature, pressure, or chemical conditions surrounding the substance.

Hot cell

A specialized shielded laboratory in which radioactive materials may be handled with the aid of remotely operated manipulators. The walls and windows of the laboratory are made of materials designed to protect workers from gamma and other radiation.

Hot run

An operational (or test) run of a chemical process and equipment using radioactive materials.

Hot settlement Pond

An outdoor basin, usually lined at the bottom with clay, in which liquids containing radioactive particles are sent to evaporate. Solids settle to the bottom, where they are adsorbed onto the clay.

Interim storage

A concept in the management of nuclear waste in which the waste is moved to an intermediary location between its point of origin and its "final" or ultimate storage location.

lodine-131

Also called radioiodine or radioactive iodine, an isotope of the element iodine, which has a half-life of about eight days. This (and other iodine isotopes) are released when the cladding surrounding spent fuel is dissolved or breached.

Ion exchange

A chemical process in which a substance dissolved in water is exchanged with another.

Ionization chamber

A device used to measure radioactivity.

Irradiate

To expose a substance to ionizing radiation in a nuclear reactor. The substance so exposed may be referred to as the target.

Isotope

Any of two or more species of atoms of a chemical element distinguished by different quantities of neutrons in their nuclei. For example, hydrogen has three isotopes: protium (one proton), deuterium (two protons), and tritium (three protons).

Linear accelerator

Adevice in which charged particles are speeded up in a straight line by successive impulses from a series of electric fields.

Manhattan Engineer District/Manhattan Project

Created by President Roosevelt in 1939, the Manhattan Engineer District of the U.S. Army Corps of Engineers was commissioned to build an atomic bomb. The effort was referred to as the Manhattan Project.

Maximum permissible dose

A regulatory limit on the radiation exposure that a nuclear worker or a member of the general public may legally receive due to radioactive releases from a nuclear power plant or other nuclear activity.

Megawatt

A measure of electrical power equal to one million watts.

Meltdown

The accidental melting of nuclear reactor fuel caused by a failure of the coolant to carry away heat.

Millirem

A unit of radiation equal to one thousandth of a "rem." See rem.

Microcurie

A measure of radioactivity equal to one millionth of a curie.

Mixed waste

Waste that contains both chemically hazardous and radioactive waste.

Moderator

A material used in a nuclear reactor to reduce the natural speed of neutrons ejected from fissioning atoms. Typical moderators are water or graphite.

Natural uranium

Uranium that has not been through an enrichment process to separate its uranium-235 isotopes. It is made of uranium-238 (99.3 percent) and uranium-235 (0.7 percent).

Neutron

An uncharged particle, a part of an atomic nucleus, having a mass nearly equal to that of a proton. One or more neutrons are present in every known element except hydrogen.

Noble gases

Elemental gases which do not generally combine chemically with other materials. They are helium, neon, argon, krypton, xenon, and radon.

Nuclear power plant

An electrical generating facility using nuclear fuel.

Nuclear energy

Energy released in a nuclear fission or fusion reaction.

Nuclear reactor

A complex device designed to contain a controlled nuclear fission chain reaction. A reactor may function for testing and experimentation (Materials Test Reactor), for the generation of electricity (any commercial nuclear power plant), for the production of weapons-related materials such as tritium or plutonium (N Reactor at Hanford), as a breeder of nuclear fuel (Experimental Breeder Reactor), for propulsion (Submarine Thermal Reactor), or as a combination of these functions.

GLOSSARY

Nuclear waste

A general term including high-level, transuranic, low-level, mixed low-level, and byproduct material. Each of these terms is further defined for regulatory purposes.

Nucleus

Center of an atom consisting of a cluster of neutrons and protons. It contains nearly all of the mass of the atom.

Plutonium

A metallic element most typically created by irradiating uranium in nuclear reactors (although small amounts have been found in nature). The fissionable isotope plutonium-239 can be used as reactor fuel.

Pressurized water reactor

A reactor concept in which water is used to cool the reactor core. It is pressurized to prevent it from boiling. Heat is transferred from a "primary" coolant pipe to a "secondary" pipe.

Primary loop

A closed system of piping through which coolant flows past the nuclear fuel in a reactor.

Prompt critical

Astate of criticality derived from the fact that a small percentage of neutrons in a chain reaction are not emitted as soon as the atom splits, but are "delayed" for as long as a few minutes. "Prompt" neutrons are emitted immediately upon fission. If a reactor goes "prompt critical," it indicates that reactivity has increased to the point that prompt neutrons alone are sufficient to maintain the chain reaction. Rapid multiplication of neutrons can occur after this point. In the SL-1 reactor accident, the

rapid withdrawal of the control rod is presumed to have brought about a state of prompt criticality, in which the chain reaction did not require the emission of the "delayed" neutrons to begin or continue.

Proton

An elementary atomic particle that is identical with the nucleus of a hydrogen atom. Along with neutrons, it is a constituent of all other atomic nuclei.

PUREX

An acronym for plutonium-uranium extraction, the name of a chemical process used to reprocess spent nuclear fuel and irradiated targets.

R&D 100 Award

Research and development awards presented by *R&D Magazine*. Only one hundred R&D innovations are recognized in the country each year.

Radiation

Energy transferred through space or some other media in the form of particles or waves. If the particles or waves are capable of breaking up atoms or molecules, then the radiation is said to be ionizing radiation.

F

An abbreviation meaning "roentgen." One roentgen (R) measures the power of gamma or X-rays to produce ionization (ie, strip an electron from an otherwise stable atom) in one gram of air.

Radioactive waste

By-products of nuclear processes which are radioactive and have no useful recyclable purpose.

Radioactivity

The spontaneous emission of particles or waves from the nucleus of an atom. The emissions may include alpha and beta particles, and gamma rays.

Radionuclide

A radioactive species of an atom. For example, strontium-90 is a radionuclide (also called a radioisotope) of strontium.

RaLa

An abbreviation for Radioactive Lanthanum, one of the fission products of a nuclear reaction. It was useful to scientists developing a plutonium bomb.

Reactor vessel

A cylindrical steel container enclosing the fuel elements, control elements, coolant piping, and other structures that support the core of a nuclear reactor.

Reflector

Part of the structure of some nuclear reactors designed to reflect neutrons back toward the core of the reactor.

Rem (or REM)

An abbreviation meaning "roentgen equivalent man," a measure of the amount of exposure (dose) of radiation that takes into account the biological effectiveness of the exposure on the particular organ exposed.

Retention basin

An outdoor basin (of any of several designs) in which liquid solutions are deposited and held pending evaporation or the precipitation of solids.

Roentgen

An international unit of measurement of gamma or X- radiation. See "R" above.

Secondary loop

In a reactor coolant system, heat carried away from the reactor core in a "primary" system is transferred to a second loop. Water in the second loop does not become radioactive and its steam is used to spin turbines for electrical generation.

Semiscale

The informal name of a scale model of a nuclear reactor operated as part of the Nuclear Reactor Safety Test Engineering Program at the NRTS/INEL. Instead of using nuclear fuel, the "core" simulated the heat of a nuclear reaction by electrical means. The device was used to study the behavior of water and steam in accidents involving the loss of coolant caused by a broken pipe.

Scram

A sudden shutting down of a nuclear reactor, usually by dropping safety rods, when a predetermined neutron flux or other dangerous condition occurs.

Shielding

Material such as lead, concrete, water, paraffin, and other materials used to prevent the escape of radiation into the ambient or working environment of people and equipment.

Spent nuclear fuel

Nuclear fuel containing fission and activation products that can no longer economically sustain a chain reaction and is withdrawn from a reactor.

Spent fuel storage basin

A pool or pit made of reinforced concrete containing water and used to store spent nuclear fuel. The water acts as a shield preventing radiation from harming workers near the pool.

Transuranic waste (TRU)

Waste materials contaminated with humanmade elements heavier than uranium, such as plutonium. Also called TRU (transuranic waste). This term also implies a regulatory definition in which the waste contains substances with a half-life over twenty years in concentrations of more than one ten-millionth of a curie per gram of waste.

Triga

The brand name of a small, low-power reactor manufactured by General Atomics for use in universities and laboratories. The reactor was in a small pool of water used as both coolant and moderator. Similar reactors are often called "trigatype" reactors.

Tritium

An isotope of hydrogen containing three protons. Tritium gas is produced in nuclear reactors and used to boost the explosive power of most modern nuclear weapons. It is also a constituent of irradiated water associated with reactor operations.

Uranium-235

A fissionable isotope of the metallic element, uranium. In nature, only 0.7 percent of all uranium mined from the ground consists of this isotope.

Uranium-238

The most common isotope of uranium. It does not generally fission, but can be irradiated in a reactor and transformed to an isotope of plutonium which does fission.

Uranium oxide

A metallic compound of uranium and oxygen, a useful form of uranium for use as nuclear fuel because it has a higher melting point than metallic uranium and can survive the high temperatures inside a reactor more readily. However, its heat transfer properties are not as efficient as those of metallic uranium.

Water-moderated reactor

A reactor concept which is designed so that water slows down the speed of neutrons ejected from fissioning atoms. Includes boiling water and pressurized water reactor concepts.

Warm run

The operation of a chemical process using materials that are slightly radioactive. A "warm" run is contrasted with "cold" or "hot" runs.

Waste storage tank

A holding tank for liquid or gaseous wastes which may or may not be radioactive.

Zirconium

A metallic element highly resistant to corrosion and used to make cladding for nuclear fuel elements. It is sometimes alloyed in small amounts in the fuel itself.

Zero power

Also called "low power," a mode of operating a reactor so that it maintains a chain reaction at extremely low power levels. It produces very little heat. Zero power reactors are used as sensitive laboratory tools to pre-test experimental loadings of test reactors and for other analytical purposes.

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